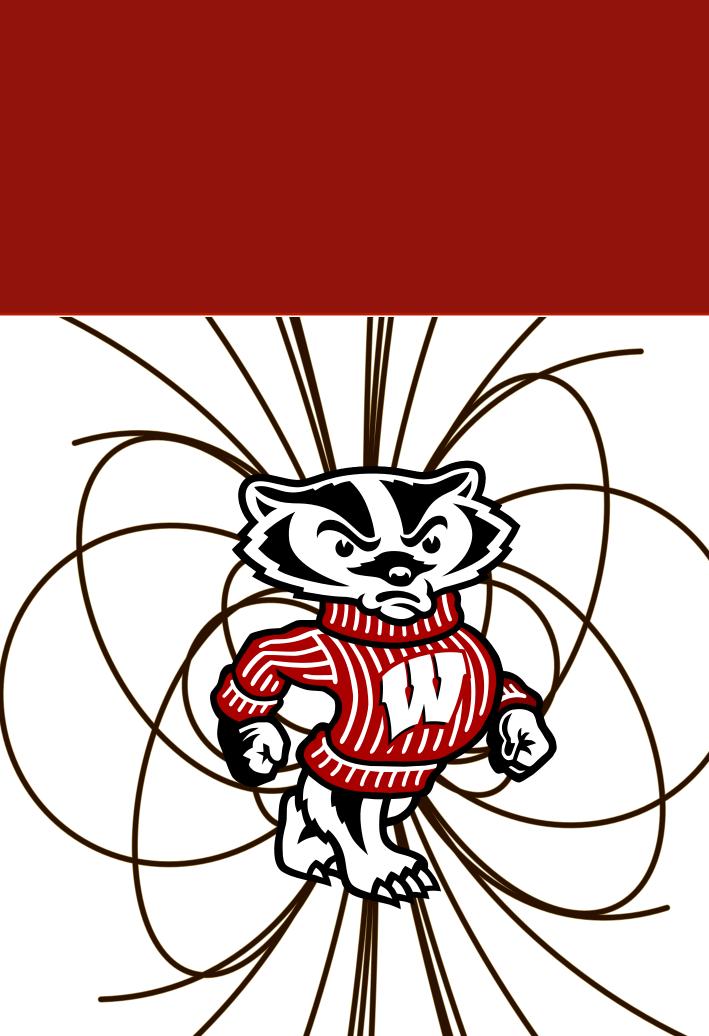
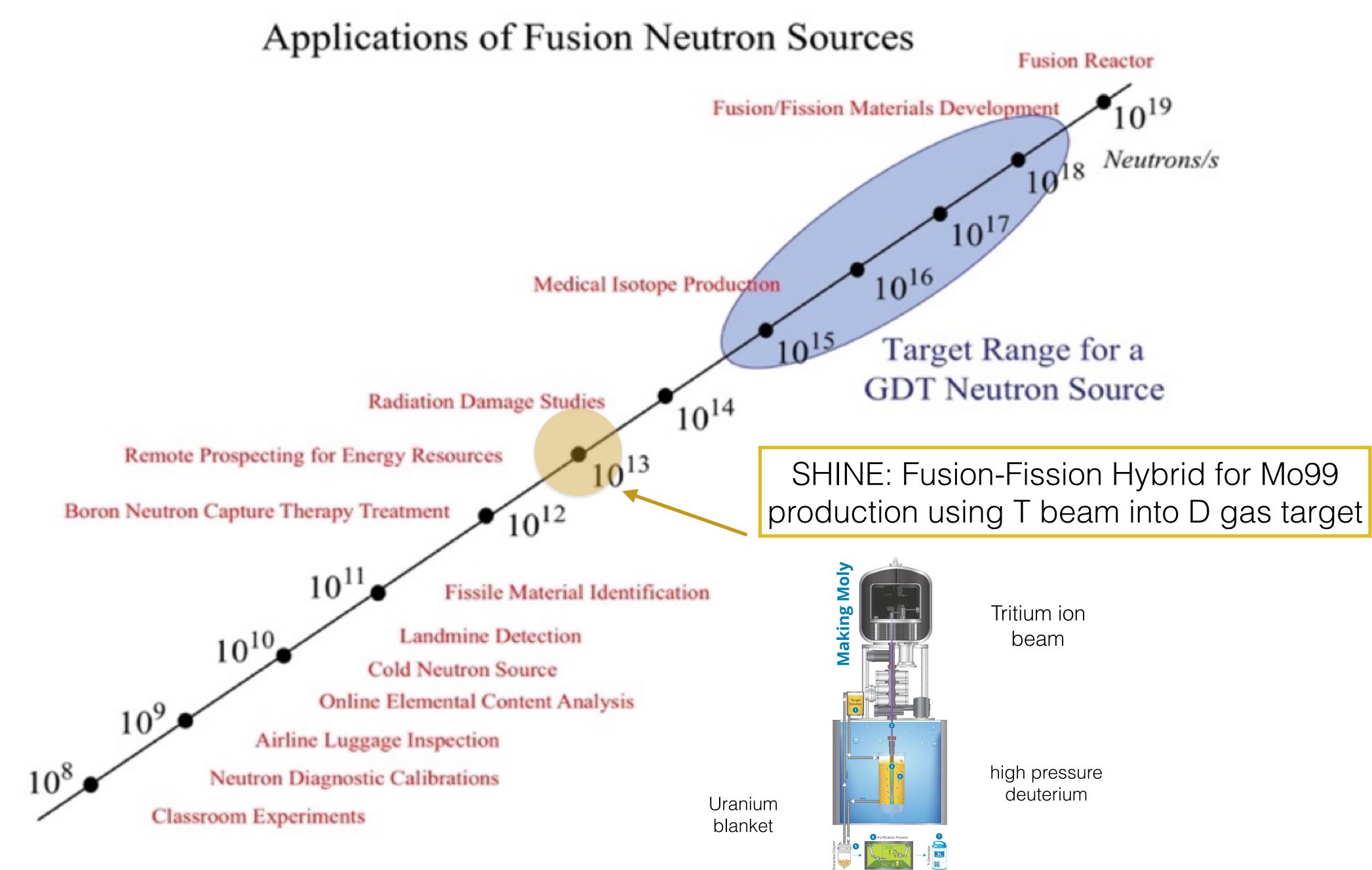
Development of GDT-based Fusion Neutron Source *On a path to fusion energy*

C.B. FOREST, J. Anderson, J. Egedal, V. Mirnov, A. Molvik, E. Peterson, J.S. Sarff, T. Simonen, O. Schmitz, J. Wallace and R. Waleffe, R. Harvey, Y. Petrov, D. Whyte, R. Mumgaard



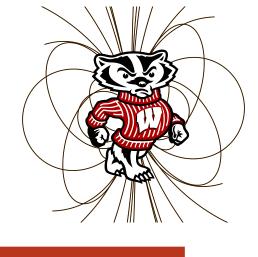






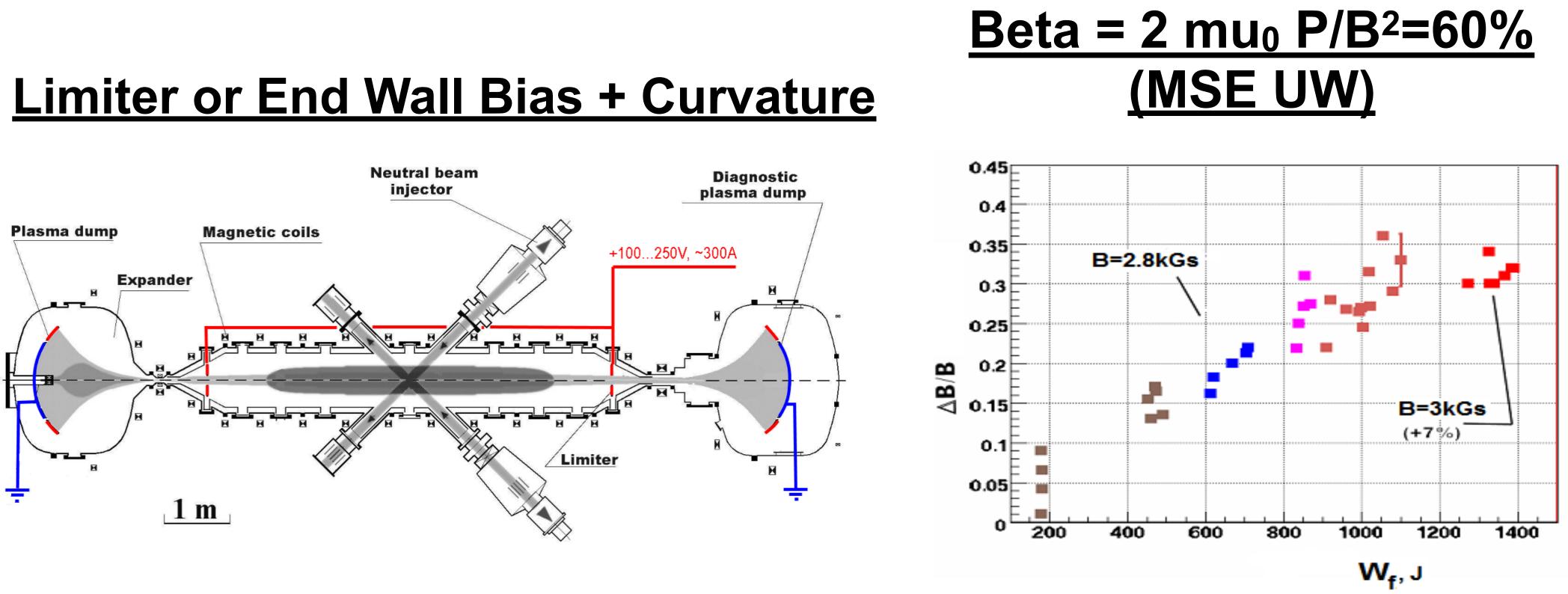
Outline

- 1. There have been recent game changing results in Gas Dynamic Trap mirror research 2.Fast ions confined in a steady-state GDT could be a very efficient neutron source for many applications (isotope production, radiography, neutron capture therapy) offers a cost-effective short cut to fusion materials and sub-component testing 3. The mirror, like tokamaks, benefits from new high field magnet technology and other technological developments (eg. high frequency steady-state gyrotrons, lithium walls, negative neutral beams)
- 4.Near term experiments and theory effort could:
 - determine if GDT neutron source is feasible
 - be test bed for high field magnets and lithium walls
- 5. The fast development path of linear systems, may only require one or two miracles beyond the GDT to improve confinement for fusion energy: there are still good ideas



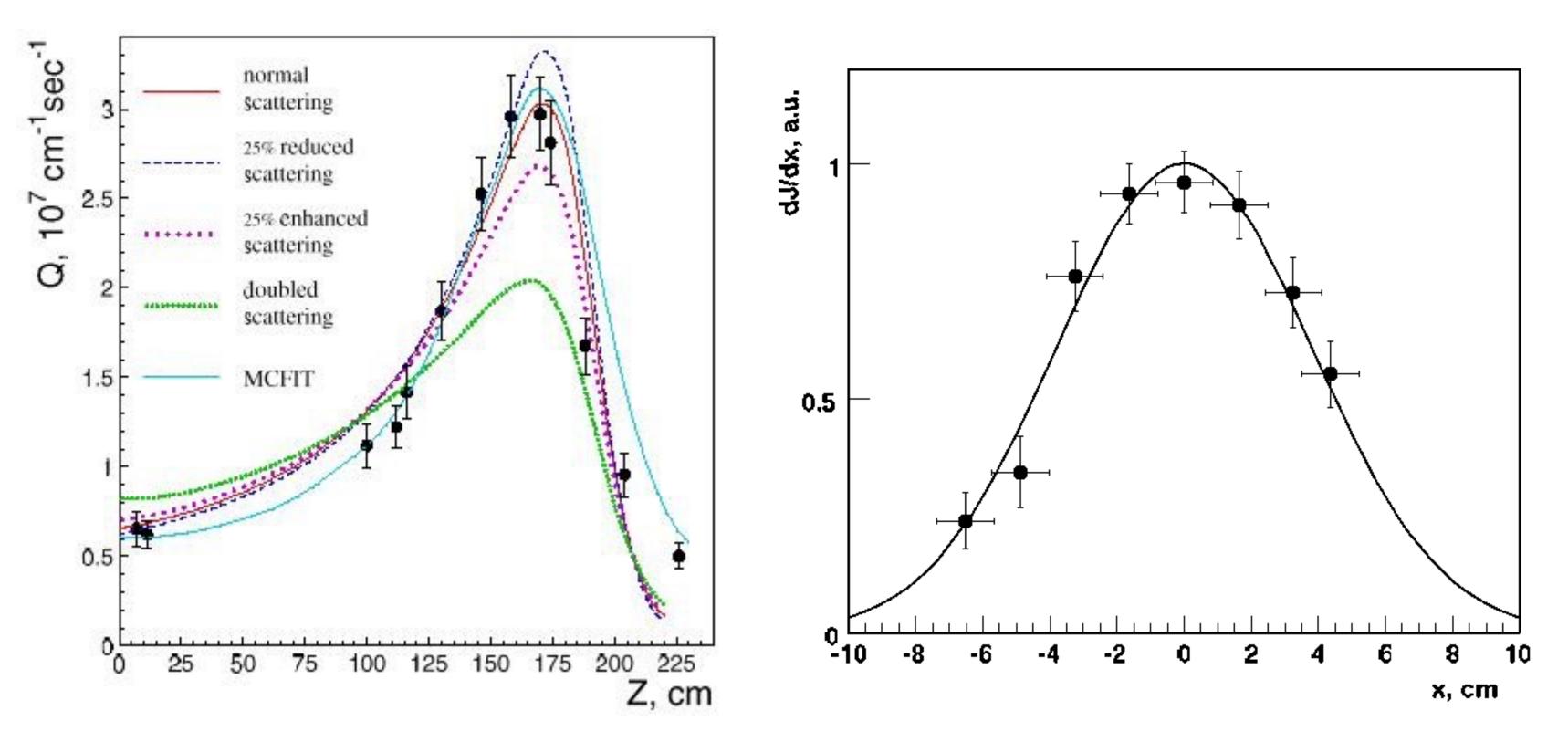
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<u>1. GDT Axisymmetric MHD Stability</u>



2. GDT D-D Neutron Production Illustrates Lack of Micro-Instabilities

Axial Profile

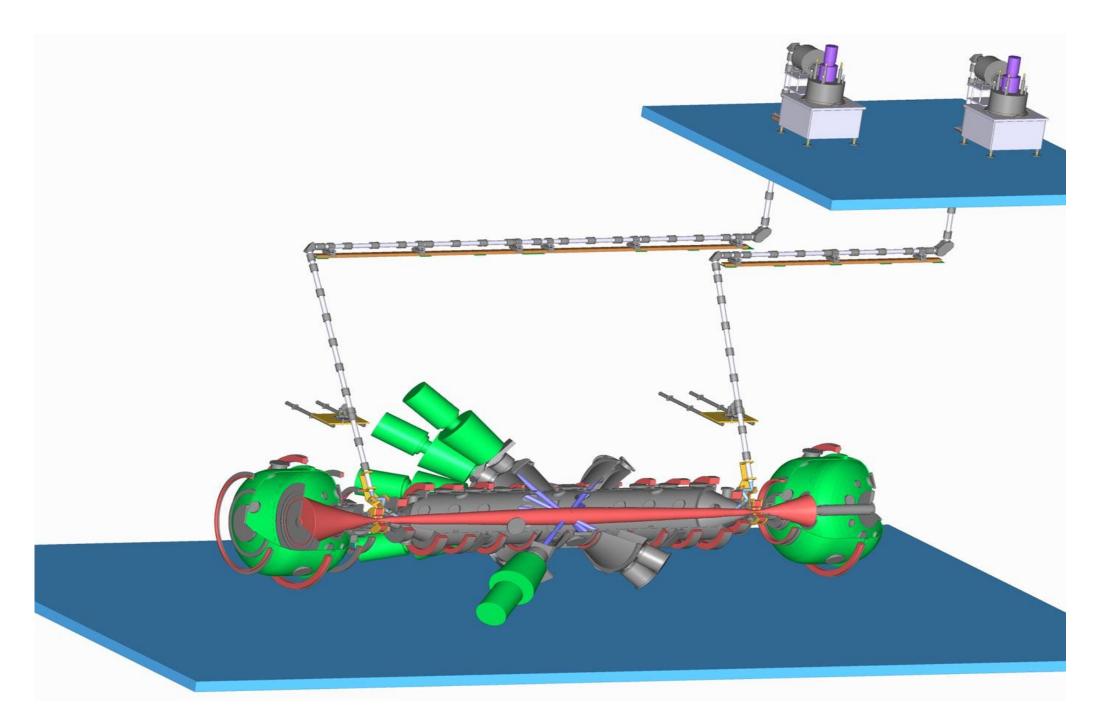


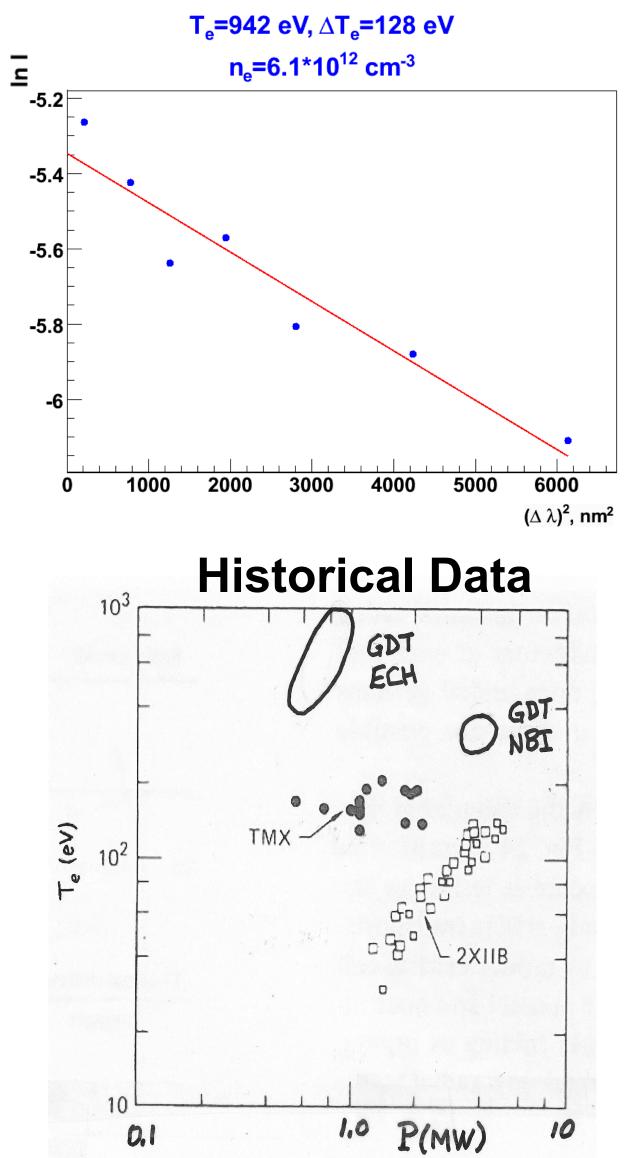
also demonstrated tandem mirror like plugging (n_{fast} >2 $n_e(z=0)$ at turning points)

Radial Profile

<u>3. GDT Electron Temperature</u> Reaches 1 keV with ECRH and Expanding Divertors

Two 0.4 MW Gyrotrons at 54.5 GHz



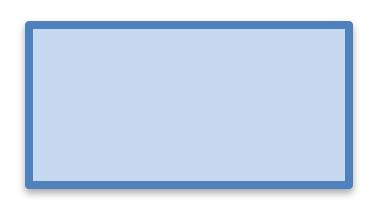


Applications depend on achievable T_e

GDT Achieved

Diverter Test Facility (≤400 MW/m²)

Neutron Source



1

Fusion-Fission Hybrid

Tandem Mirror Fusion Power Plant



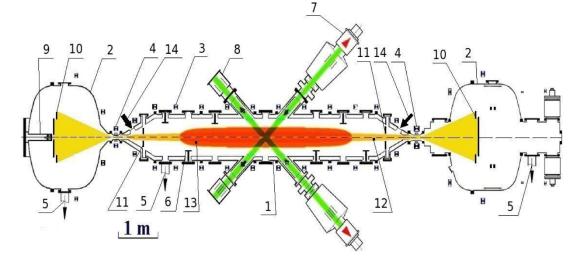
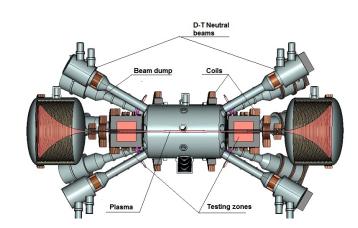
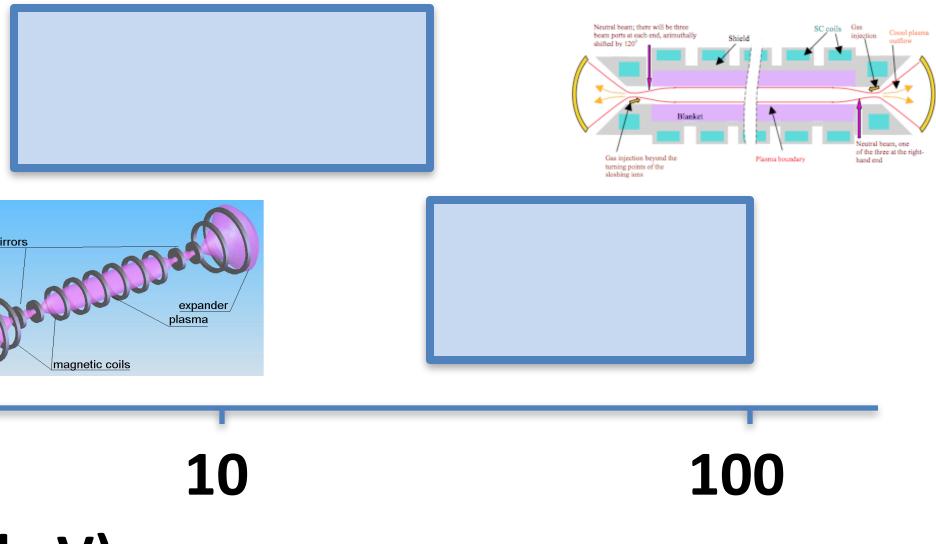


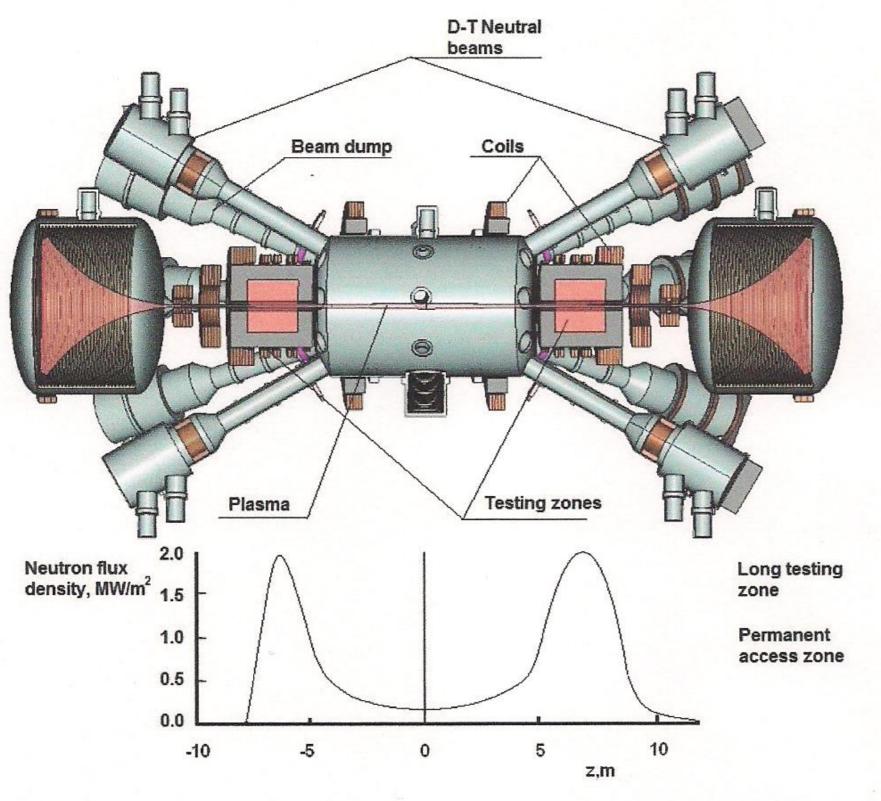
Figure 1. Schematic of the GDT device: 1 – central vacuum vessel; 2 – end tanks; 3 – central solenoid coils; 4 – mirror coils; 5 – vacuum pumping ports; 6 – titanium evaporators; 7 – neutral beam injectors; 8 – neutral beam dumps; 9 – initial plasma source; 10 – sectioned end-plates (central parts are grounded, external rings are biased with approximately +300 V); 11 - biasing limiters; 12 – warm plasma column; 13 – hot ion column; 14 – ports for launching of microwave radiation.





DTNS: A Russian Design for a Neutron Source

A MW of Fusion Power for Weeks Neutron Flux ~ 2 MW/m² Test Area ~ 1 m²



A.A. Ivanov, Fusion Science & Technology 55 2010 A.A. Ivanov and V. V. Prikhodko, PPCF 55 2013 and references therein.

Gas Dynamic Trap Accomplishments:

- axisymmetric MHD stability, $\beta \sim 60\%$
- Te ~ 0.9 keV
- $B_c \sim 0.3T$, R < 40
- 25 kV beams, 5 ms with classical slowing down and sloshing ions

Scales to neutron source with higher B, higher NBI energy, higher ECH frequency

- Te = 0.65 keV
- $B_c = 1.3 \text{ T}, R = 10$
- 65 kV beams (DT for high flux)
- 2 MW/m² (4x ITER flux)

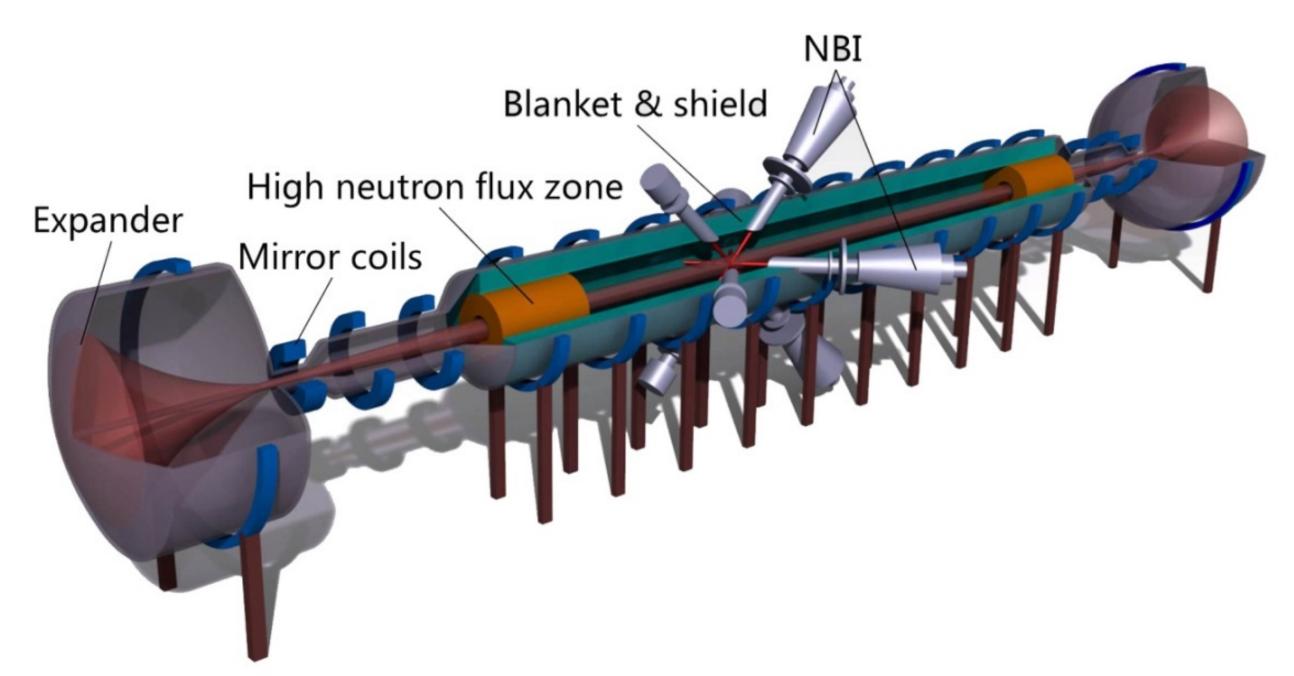
50 MW input! (low Q at 15m device) A GDT reactor would be ~ 1km long

Tritium consumed 200g/fpy

- No Tritium breeding needed
- $10 \text{ dpa} = 1 \text{ MW/m}^2 \text{ fpy}$ \bullet



Initiate an International Mega-science Project International Fusion Volumetric Neutron Source (FVNS)



International Cooperation:

- Design and Licenses in ~5 years Construction in ~5 years Operation in ~20 years Construction Cost is ~1Billion \$

- **Design target for FVNS**
 - **Neutron spectrum:** fusion neutron
 - Availability: > 70%
 - **Operation:** quasi-continuous
 - **Tritium consumption rate: <200g/year**
 - Neutron flux and test volume:

 $\geq 2MW/m^2$ (~10L); $\geq 1MW/m^2$ (~100L); $\geq 0.5MW/m^2$ (~1 m³);



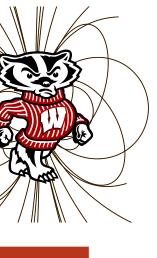


Gaps (shortcomings) remaining from **Budker GDT Experiment**

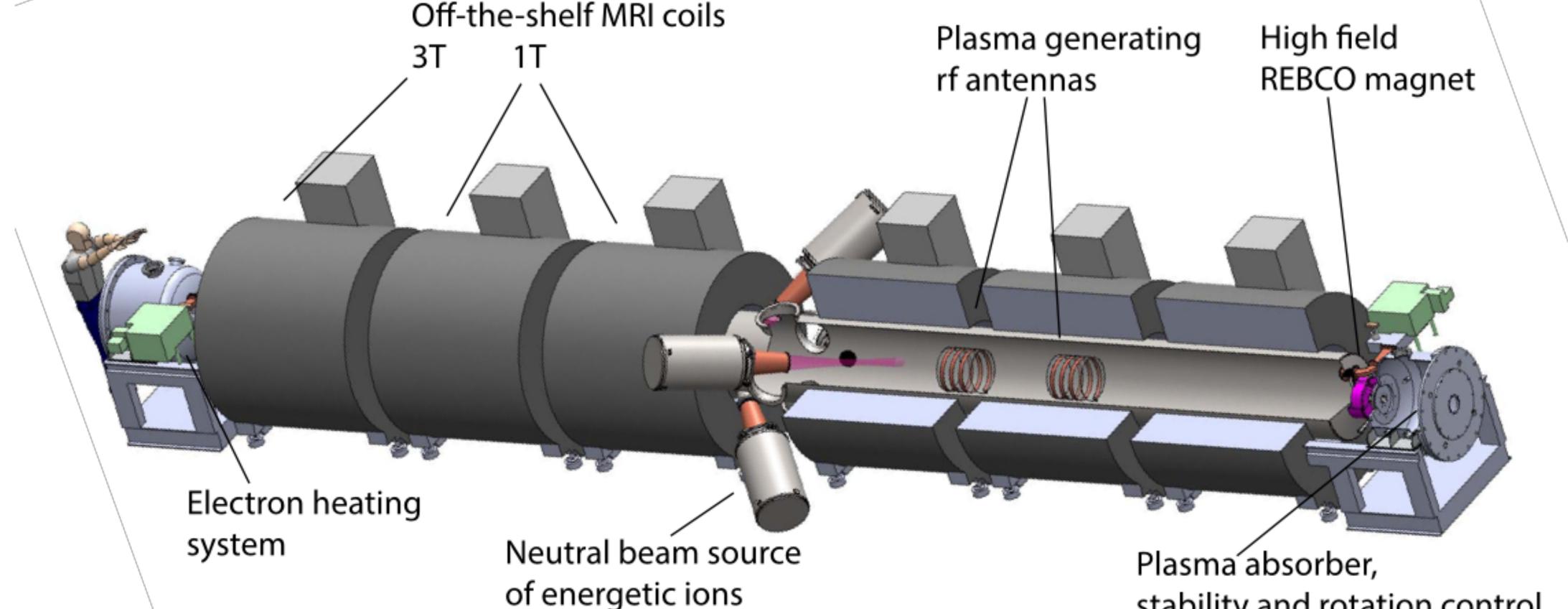
- short plasma pulse (5 ms)
- low field (~2 kG)
- low energy neutral beams
- low frequency (and therefore low density) ECH less than optimized pumping and fueling (heavy use of Ti gettering)

Advanced Concept Exploration at Mid-Scale is a necessary next step

- longer pulse (>50 ms to reach steady-state fast ion distribution) higher field ($B_c=0.5-1$ T, $B_{plug}>20$ T for fast ion and high density)
- full energy neutral beams (80 keV)
- high frequency (and therefore high density) ECH
- Optimized pumping and fueling (Li walls)



A cost-effective next step GDT would be a high-field, medium pulse (0.1-2 sec)



- Reliable, low-cost central cell magnets using **commercial** MRI-industry magnets (1-3T @\$400k)
- Two compact, 20 T high-field mirror plug coils (simple planar REBCO coils)
- Novosibirsk-developed NBI injection for fast ions (2 MW, eg MST + TriAlpha beams)
- Expanding-field Li diverter for MHD stability and pumping
- HHFW for heating ions at turning points
- ECH for high density plasma formation and T_e control (2 MW at 110 GHz)

stability and rotation control

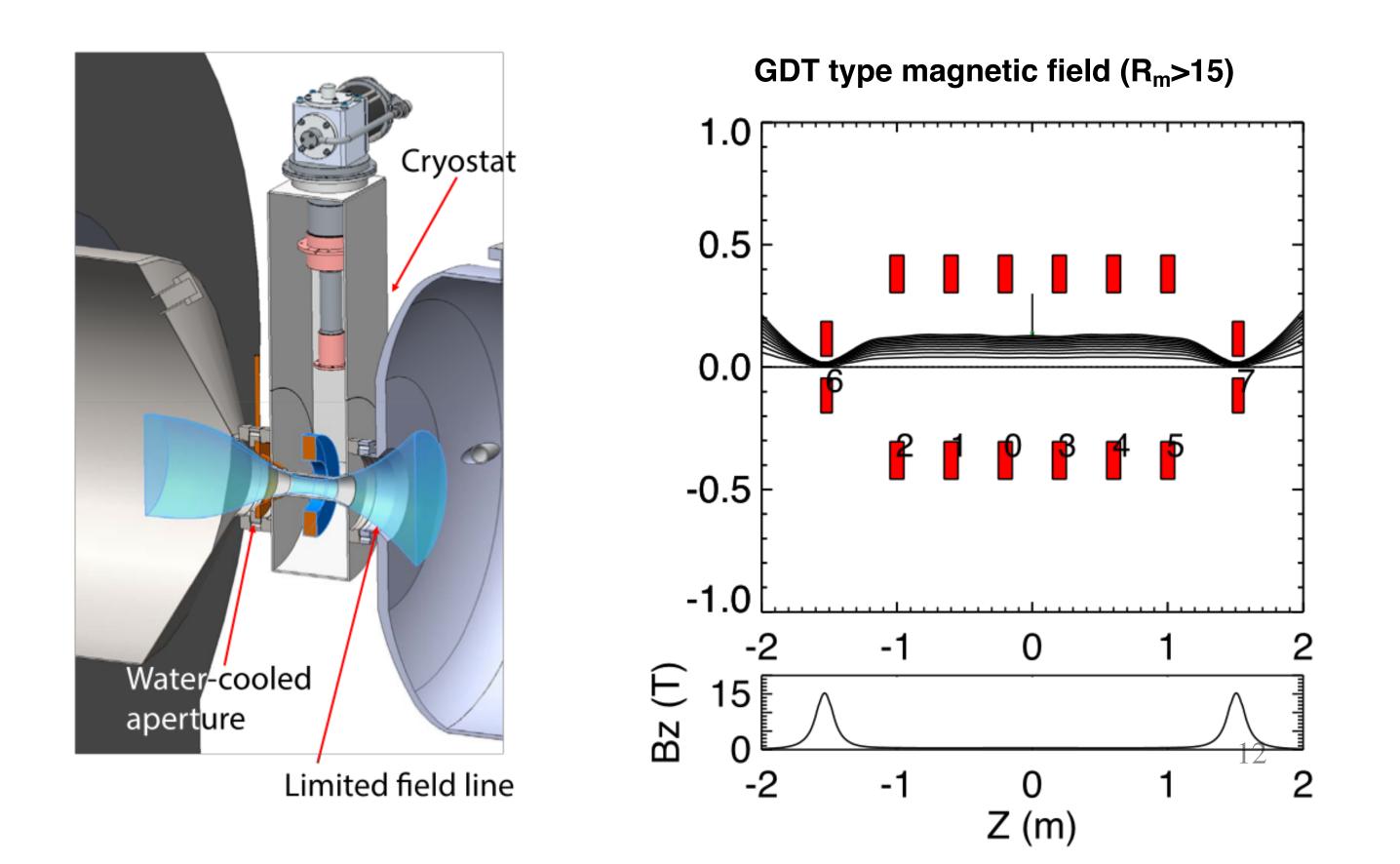
a= 10 cm $P_{NBI} = several MW$ $P_{ECH} = 1 MW$ $f_{ECH} = 110 \text{ GHz}$

Investment of internal funding by UW-Madison is being used to construct a GDT prototype device featuring HTS magnets

- The Rotating Wall Machine is being repurposed as a prototype mirror device
 - Will produce high density target plasma
 - Expander physics with Li end walls
 - Rotation/ biasing with LaB₆ cathodes
 - Adequate diagnostic set on hand

Prototype device in UW Physics Dept.





REBCO based coil under development

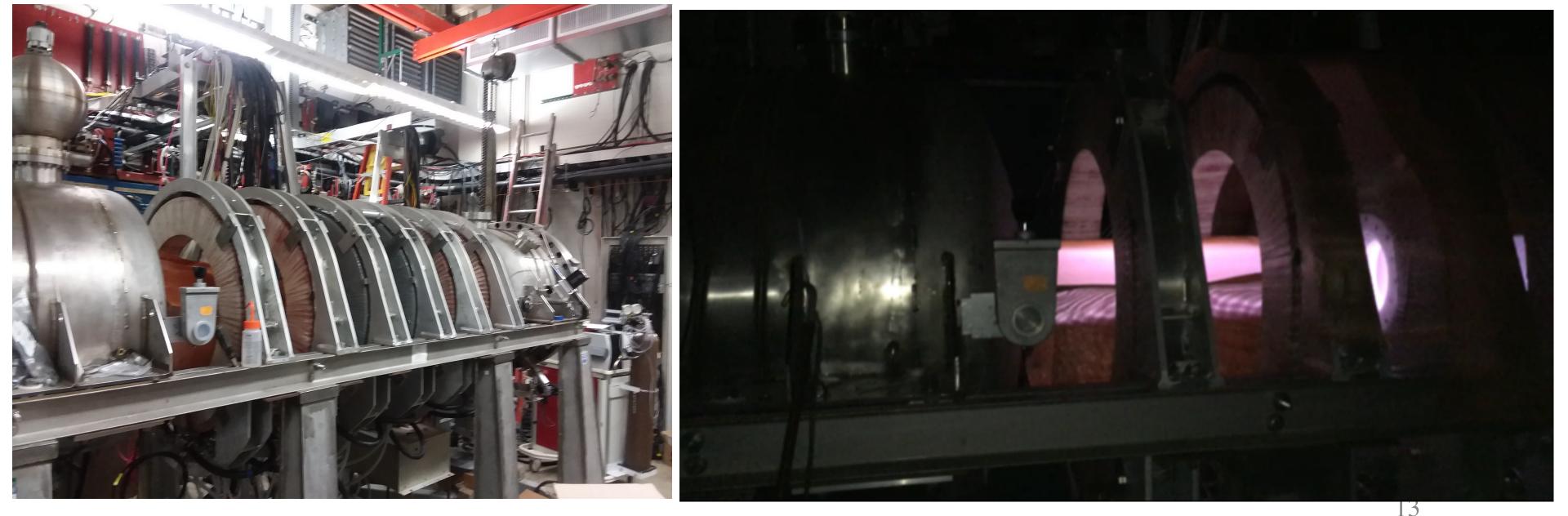
(collaboration with both General Atomics and CFS)

- Will help drive use of HTS for plasma confinement

magnets

Physics goals for proto-type experiment:

- Demonstrate long pulse MHD stability in GDT geometry
- Understand and control Te
- begin sloshing ion studies
- Explore use of liquid Li in end-cells; compatibility with GDT plasma fueling and exhaust



Proto-type device in UW Physics Dept.

Investment of internal funding by UW-Madison is being used to construct a GDT prototype device featuring HTS

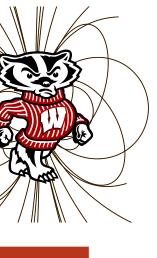
First plasma 5/30/18, central solenoid only

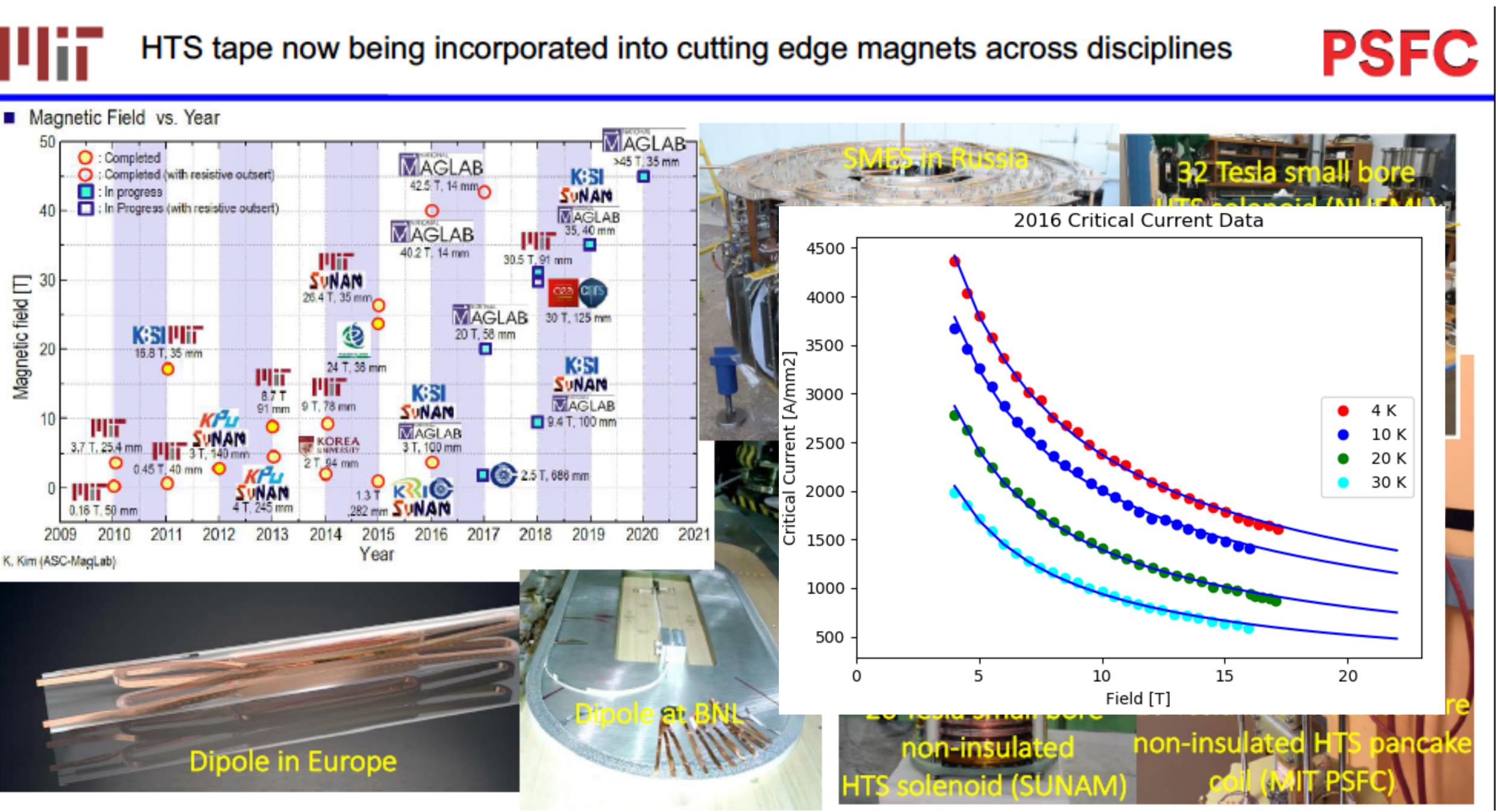
HTS for Mirrors

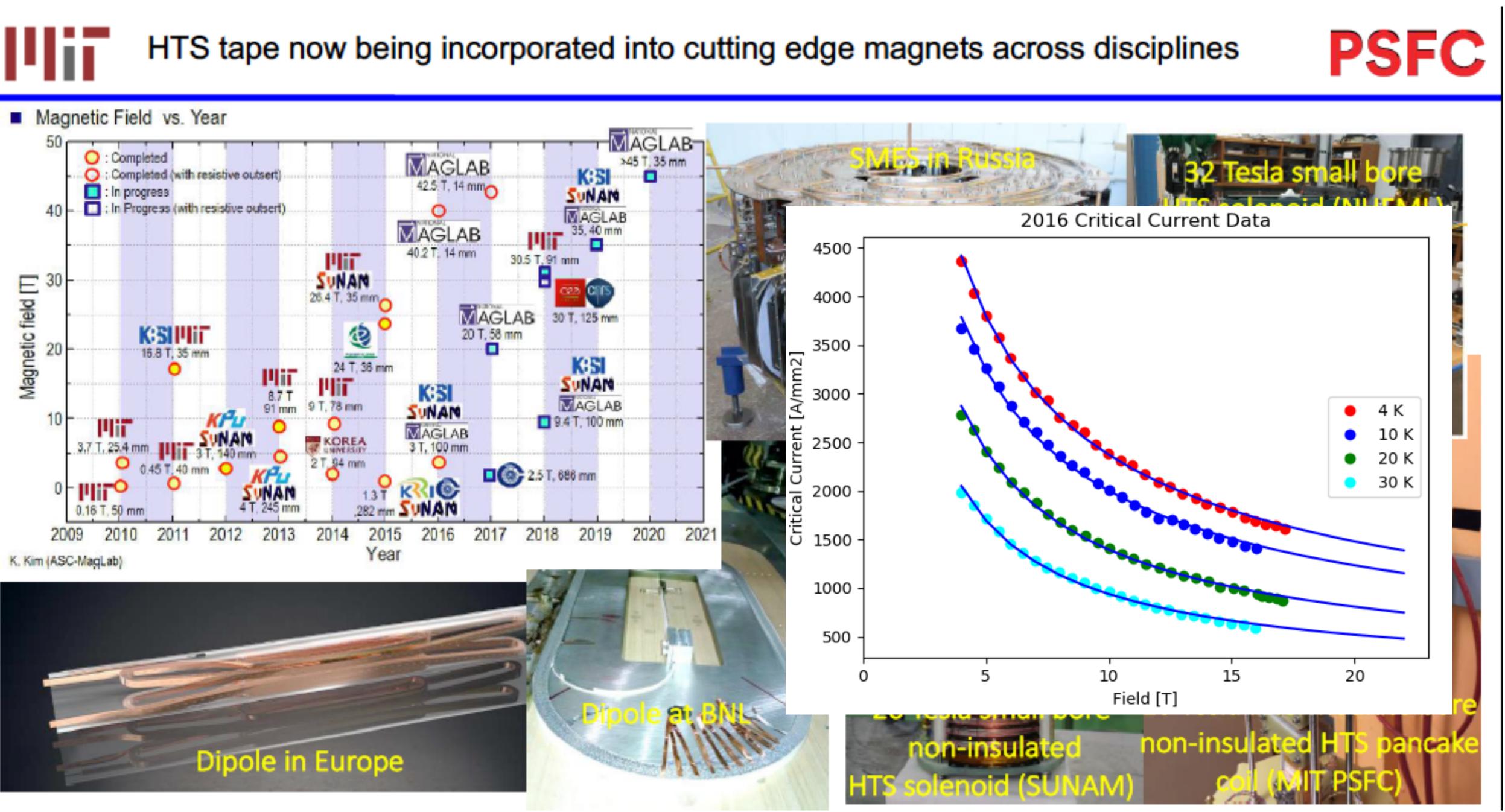


- 1. Higher Field
- 2. Higher mirror ratios

3. magnet winding pack is can be different





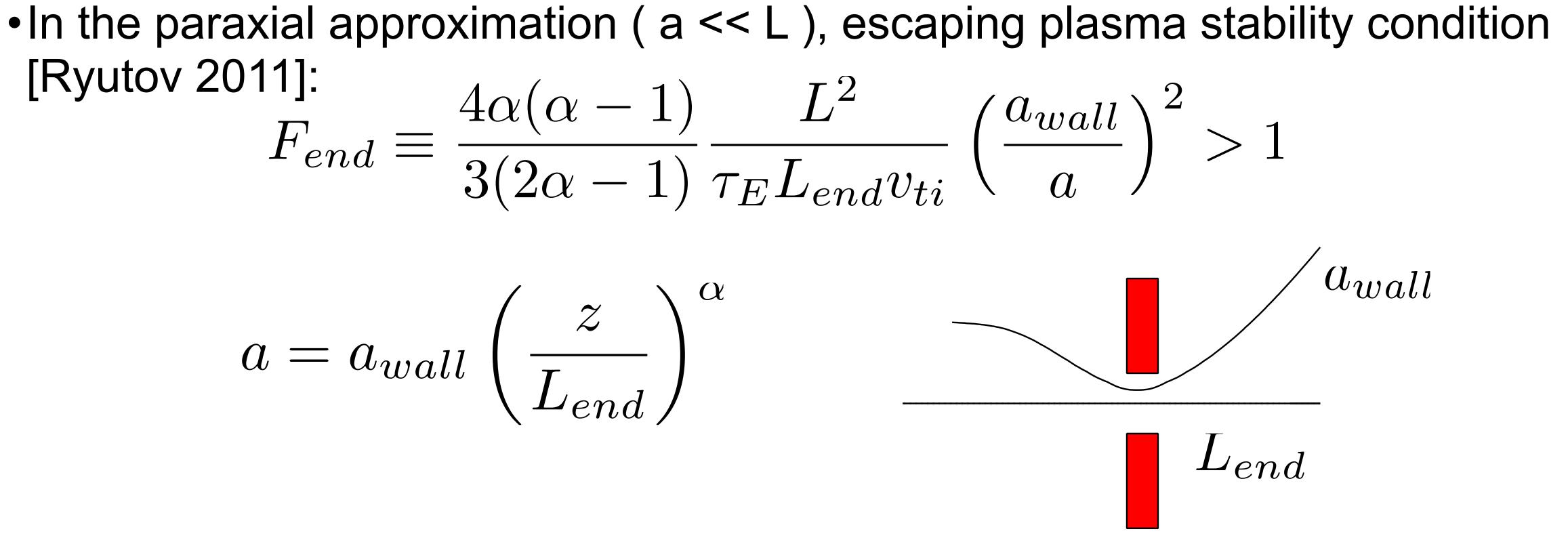




[Ryutov 2011]: $F_{end} \equiv \frac{4\alpha(\alpha-1)}{3(2\alpha-1)} \frac{L^2}{\tau_E L_{end} v_{ti}} \left(\frac{a_{wall}}{a}\right)^2 > 1$ $a = a_{wall} \left(\frac{z}{L_{end}} \right)^{\alpha}$

- •Necessary condition on field reduction: $\alpha > 1$
- design, length of device, etc, but $\alpha > 1$ necessary

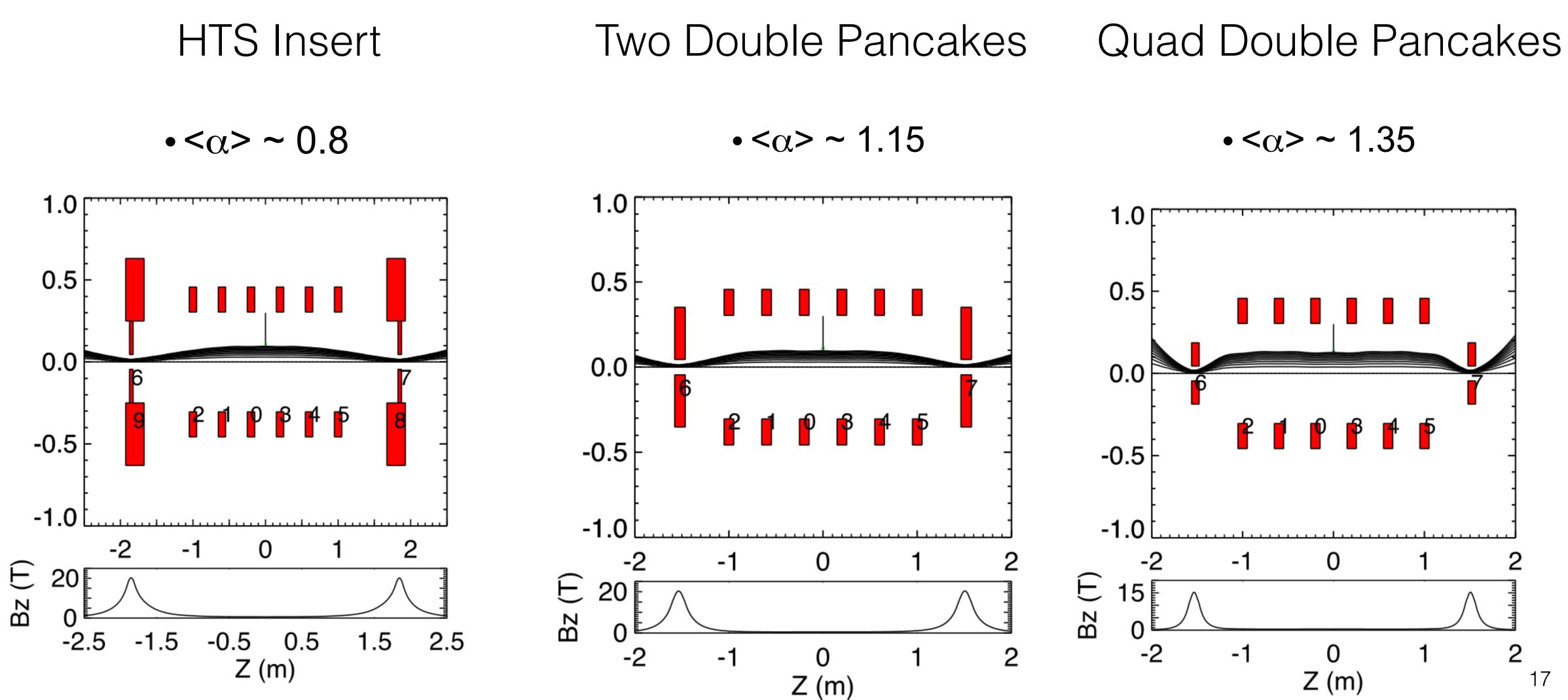
Field fall-off in expander region key for MHD stability



•Full calculation, including fast pressure required. Some freedom in end cell









The Short-Fat Mirror as Alternative to GDT?

1.Simplest Mirror

- •Planar small bore REBCO coils
- standalone or potential tandem mirror end cell
- 2.Axisymmetric MHD Stability
 - •MHD stable to m=1 from flux expansion
 - •FLR stable for $m \ge 2$ (?)
 - •Rosenbluth-Hinton stability from sloshing ions (?)

3.Transport

- high mirror ratio (~100) and high beta •self-plugging from sloshing ions (?)
- 4.Large diameter minimizes NBI shine through

Applications: •10¹⁵ DD neutrons/s for Mo99 nuclear materials testing basic physics of collision less anisotropic MHD •end cells for tandem



The Ryutov Non-Paraxial Mirror for Tandem End Cells Theoretically MHD stable at $\beta \sim 1$ for m=1 conjecture: FLR stabilization for m=2,3,4.

Flux expansion stabilizes m=1

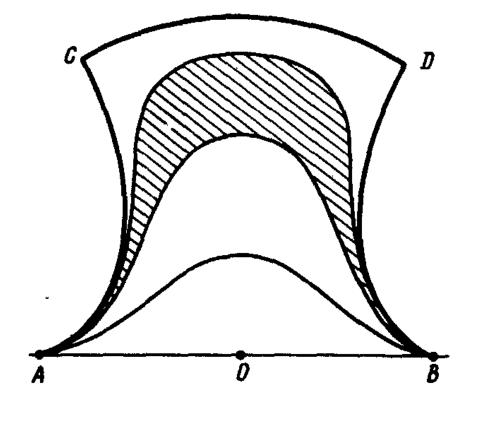


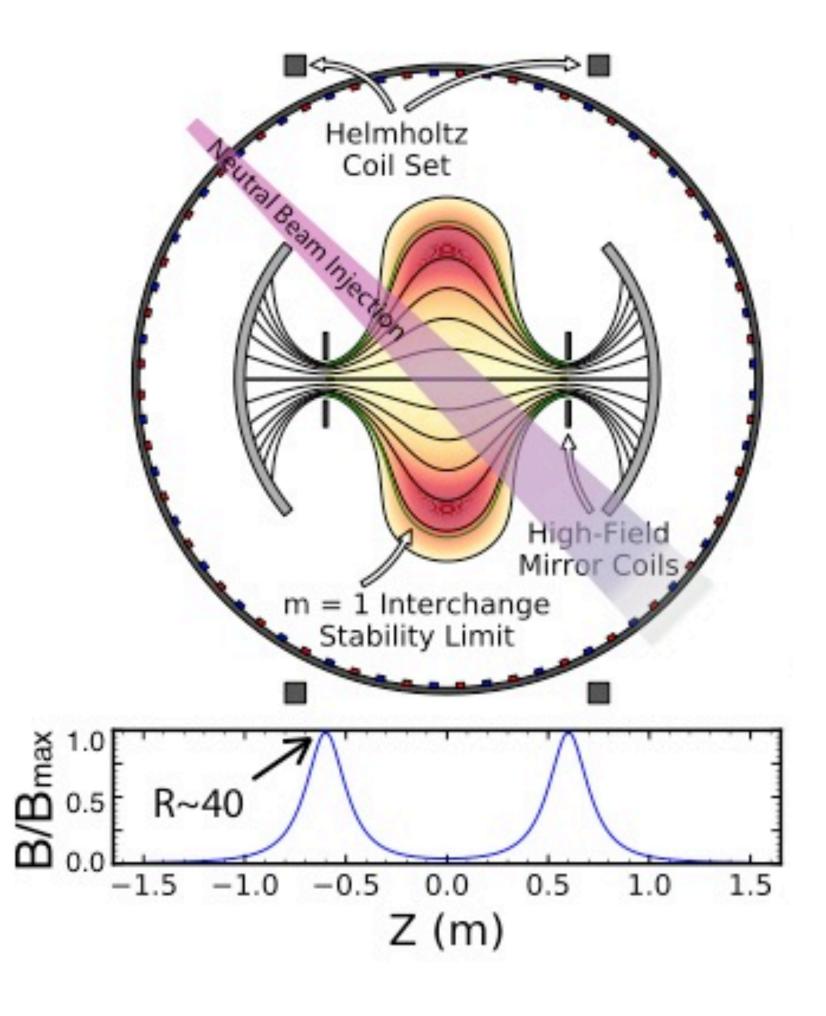
FIG. 1. Magnetic lines of force. The heavy line is the separatrix. The magnetic field vanishes at points C and D. The hatched region is the "stability ring."

36 JETP Lett., Vol. 42, No. 1, 10 July 1985

D. D. Ryutov and G. V. Stupakov



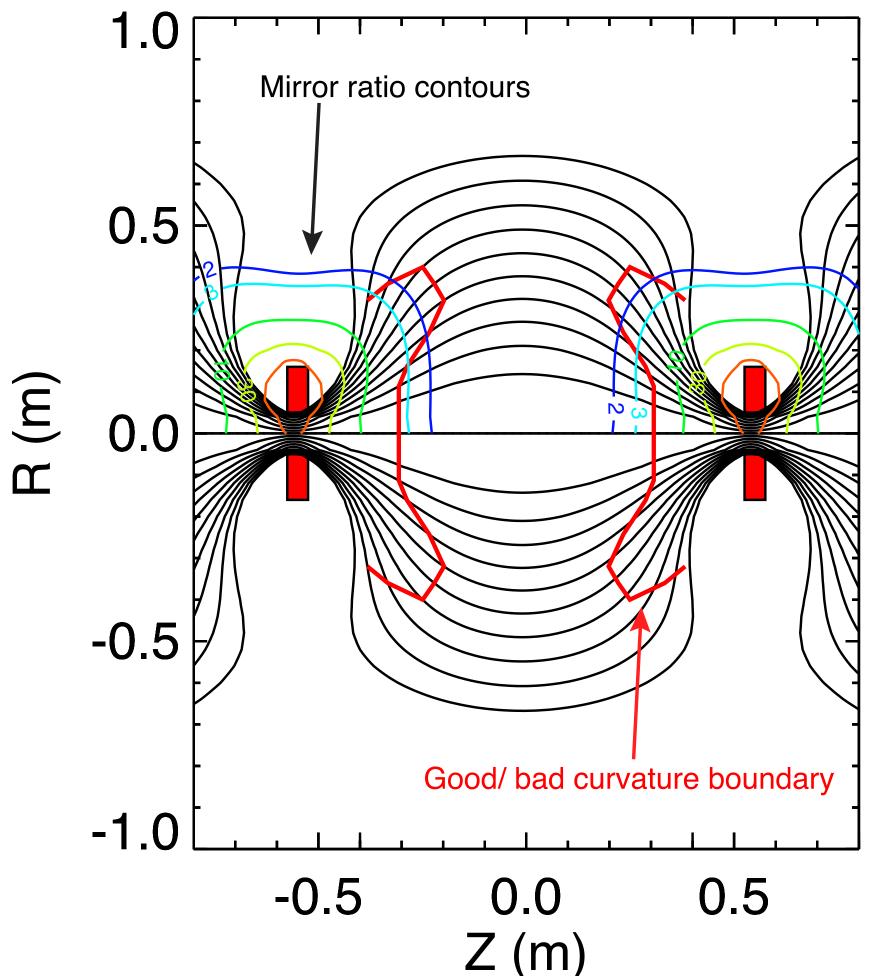






Can sloshing ions be used to stabilize plasma and selfplug short-fat mirror?

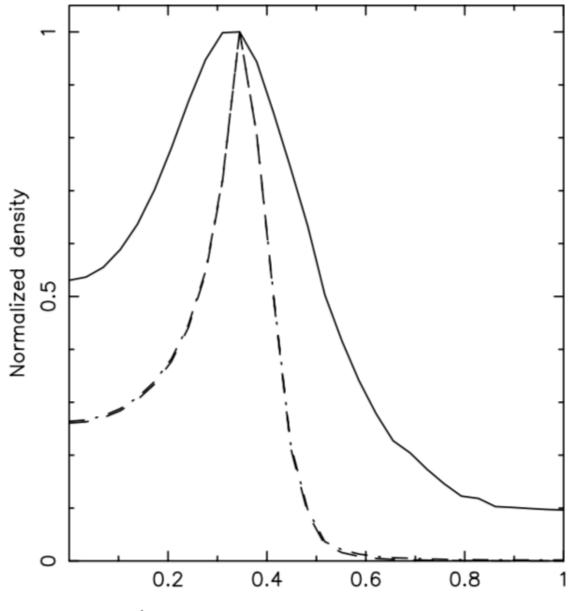
Following Rosenbluth/Hinton, pressure weighted stability when



Turning points controlled by angle of injection: NBI injection at angles with turning points in good curvature (probably ~30 degrees)

 $\int \frac{d\ell}{B^3 R^2} \vec{\kappa} \cdot \nabla \psi \left(p_\perp + p_\parallel \right) > 0$

Density as a function of axial dist on the field line

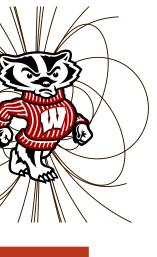


Z/Zmax axial distance on the field line

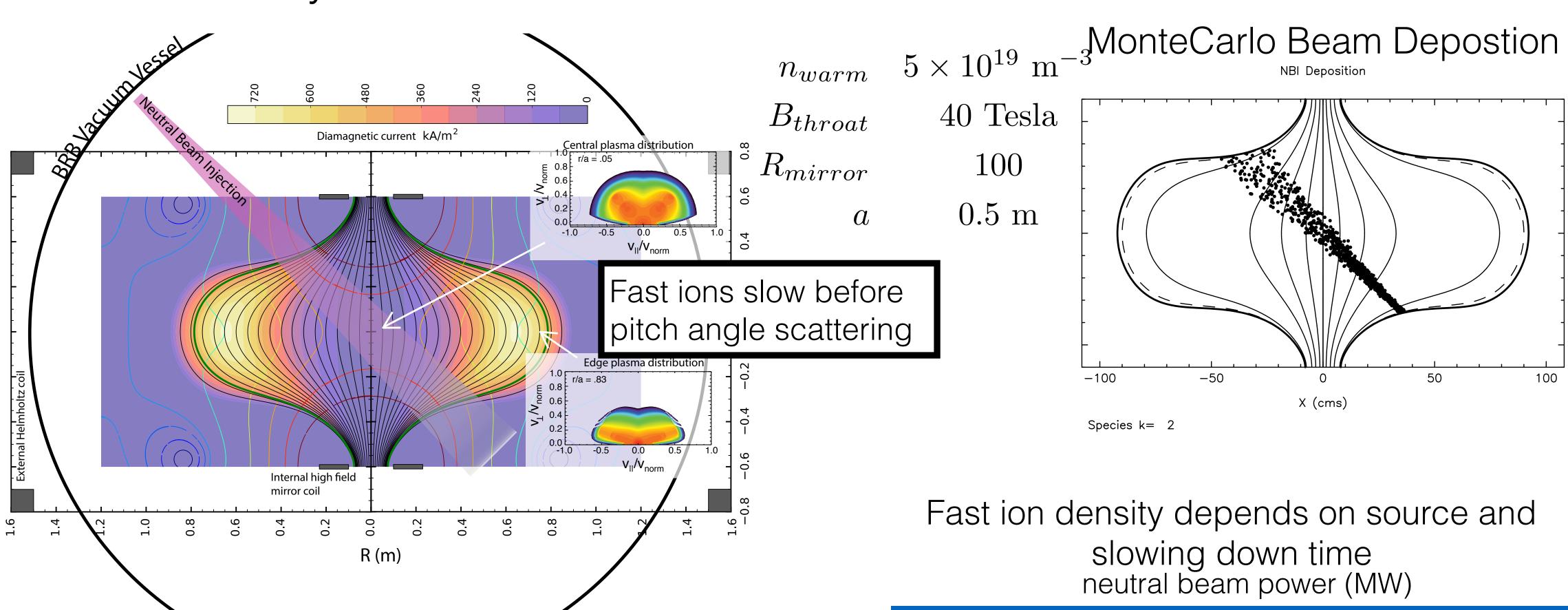
(curves are normalized to a maximum of 1.) (Line order: full,dashed,dash-dot,dotted)

species 1 Ir= 1 r/a=1.10E-05 n= 10 time= 1.0000E-01

lower egy = 0.00E+00 kev; upper egy = 6.00E+00kev maximum density on this curve = 1.92469E+11/cm**3 lower egy = 6.00E+00 kev; upper egy = 2.40E+02kev maximum density on this curve = 1.11197E+13/cm**3 lower egy = 0.00E+00 kev; upper egy = 2.40E+02kev maximum density on this curve = 1.12479E+13/cm**3



CQL3D Analysis of Neutron Yield



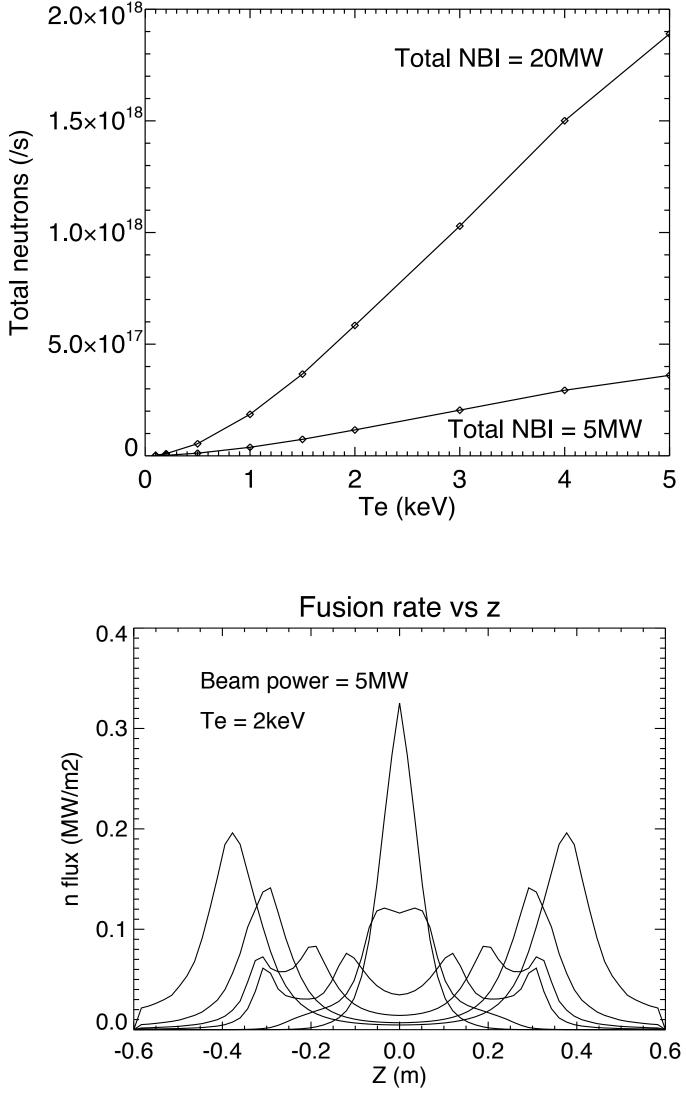
 $\frac{\partial P_{\parallel}}{\partial B} =$ Equilibrium Condition:

Te (keV)	1	2	5	10	20	
0.1	0.1625	0.2	0.36875	0.575	0.8	1.25
0.2	0.35	0.36875	0.6125	0.8375	1.2125	1.934
0.5	0.504687	0.725	1.175	1.625	2.30469	3.659
1	0.8	1.1375	1.83125	2.6	3.67812	5.862
1.5	1.025	1.49375	2.35625	3.36875	4.77969	7.587
2	1.23125	1.75625	2.7875	4.025	5.675	9.03
3	1.55	2.225	3.47187	4.925	6.95	11.17
4	1.74688	2.525	3.96875	5.61875	8	12.70
5	1.8875	2.70312	4.2875	6.125	8.675	13.82

densities in 10¹⁹ m⁻³



Parametric dependence of DT neutron yield



20,30,45,60,75,88 degrees from normal

