

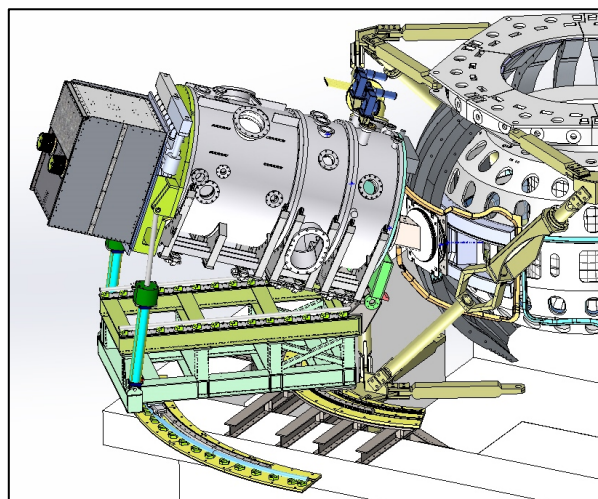
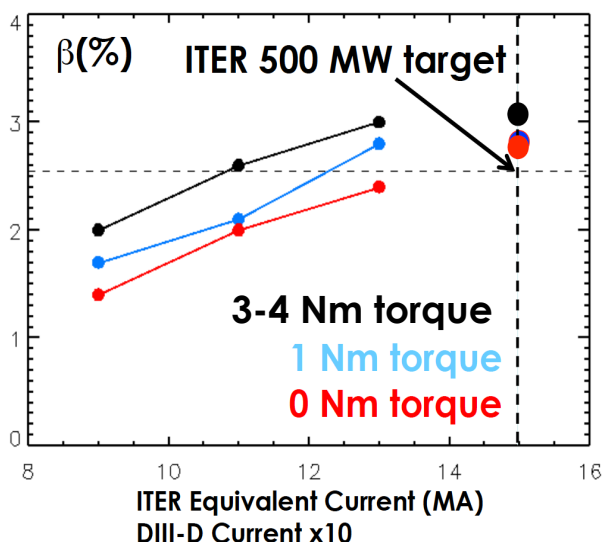
DIII-D Research Goals: Develop the Physics Basis for ITER Q=10 and Future Steady-State Tokamaks

Pursue low torque, transient-free stable cores with high fusion gain

Develop advanced current profiles for high β_N and high bootstrap fraction

Optimize H&CD for steady-state burning plasma operation

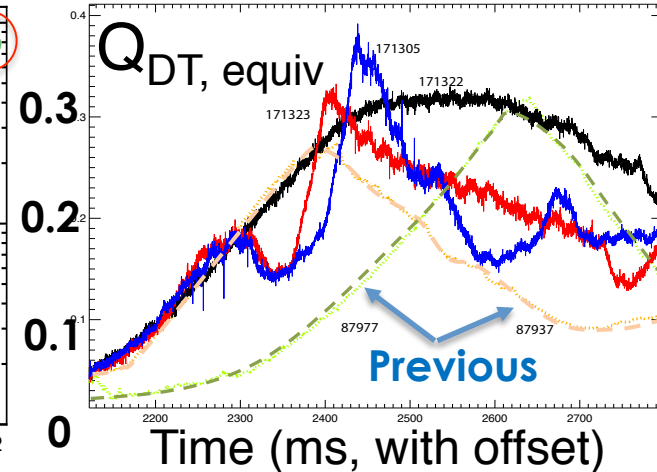
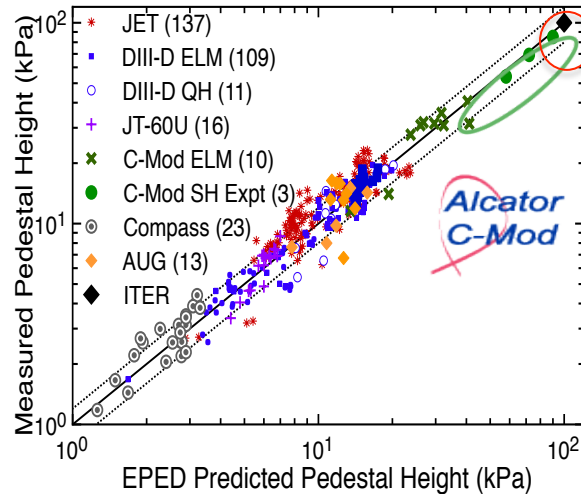
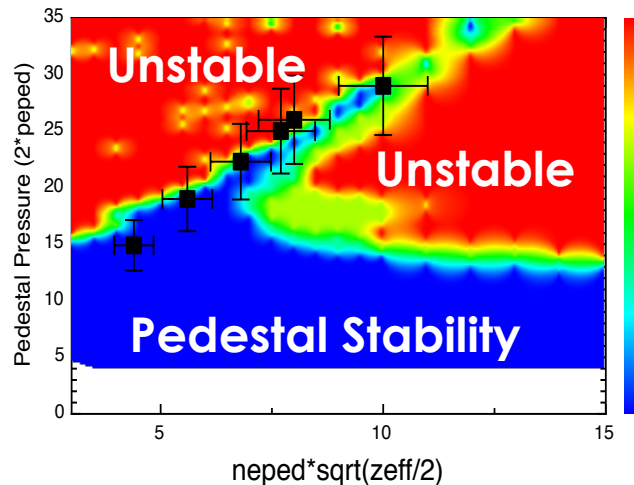
— ECCD, Helicon, and HFS-LHCD



Recent Super H-Mode Experiments Yield Large Gains in Fusion Performance

Co-torque ELMing H-mode

EPED Predictions Bt=2.1T, 1.6MA, data from rise of 171322,23

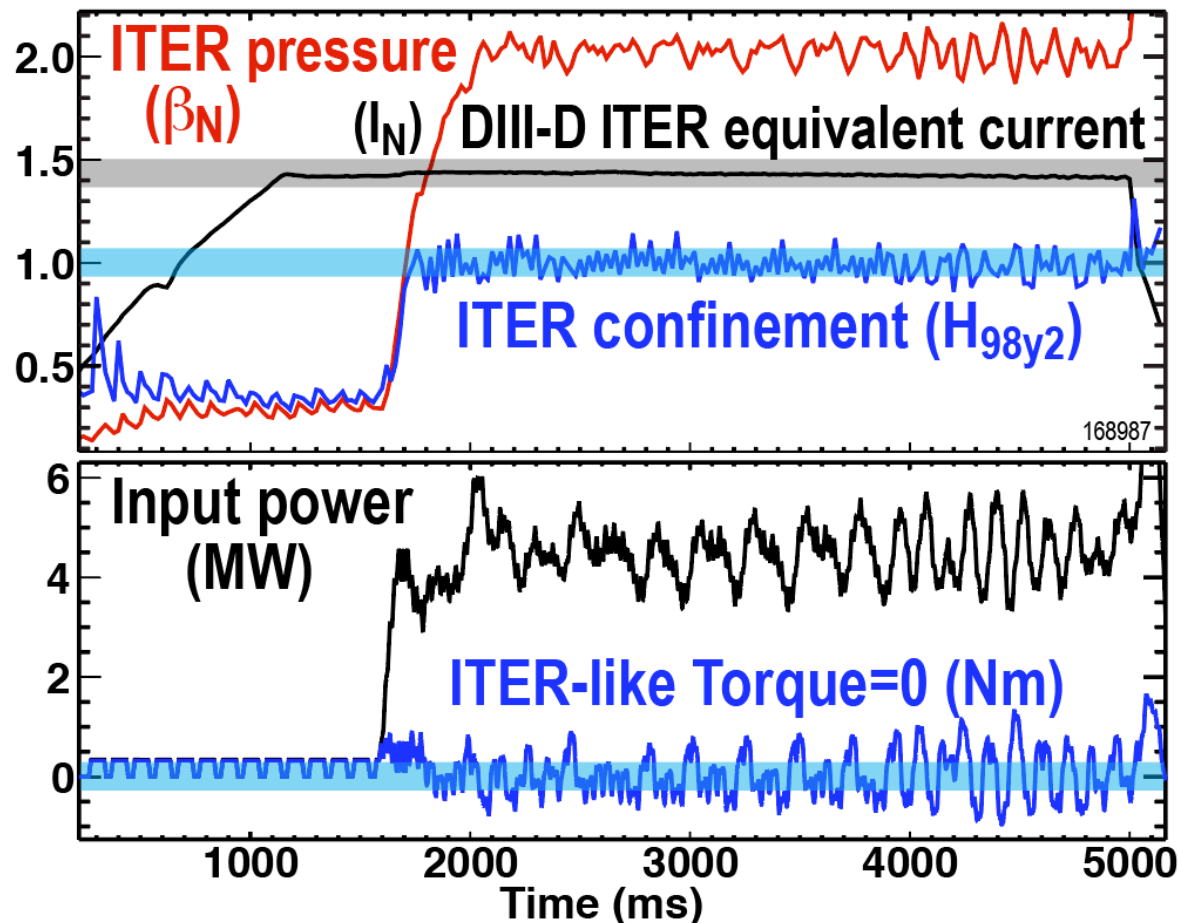


- Pedestal stability model led to Super H-mode development with high pedestal pressure and corresponding higher global energy confinement
- Super H-Mode experiments on Alcator C-Mod reached ITER-like record pedestal pressure (~80kPa) on last day of operations
- Super H-mode experiments on DIII-D yield equivalent $Q_{DT} = 0.35$

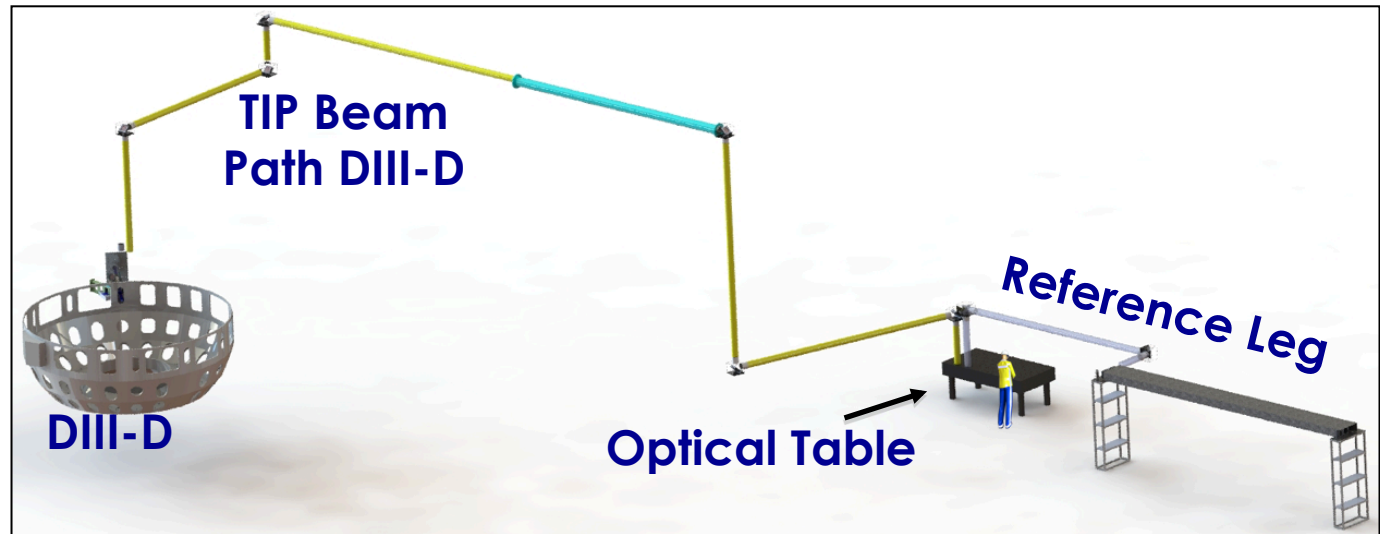
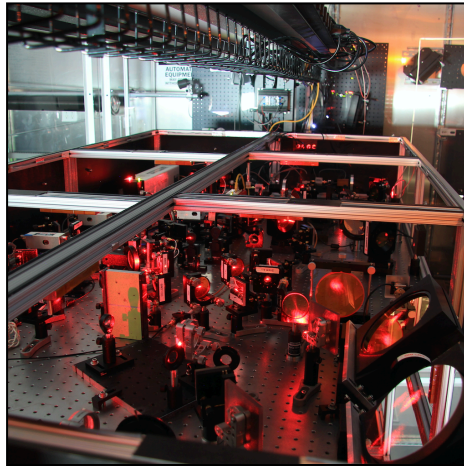
Stable Zero torque ITER Baseline Scenario Demonstrated for the first time in DIII-D

DIII-D discharges robustly simulate the first scenario in the ITER plan

- Matched ITER shape & q_{95}
- Achieved ITER required confinement $H_{98y2}=1$
- Obtained fusion gain factor sufficient for $Q=10$
- All at ITER-similar torque in repeatable conditions



US Contributions to ITER: Successful Test of Prototype Toroidal Interferometer Polarimeter on DIII-D

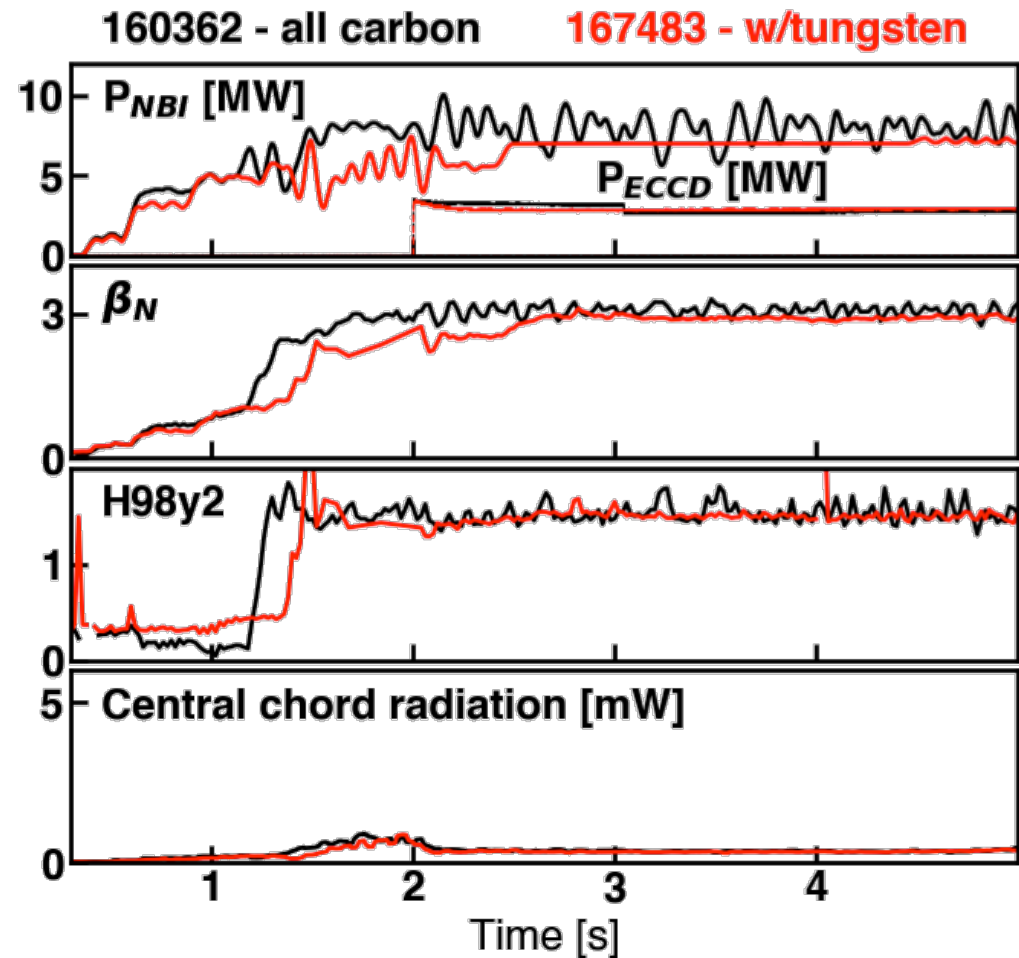


- Joint effort: UCLA-GA-PSI Two-color system ($10.6\mu\text{m}$ and $5.2\mu\text{m}$)
- Full-scale ITER prototype with 110m roundtrip path to DIII-D and back
- System includes matching 110 m reference leg
- Active feedback is used to maintain alignment

**General Atomics Is Manufacturing
the Coils for the ITER Central Solenoid**

On-axis ECCD Maintains Performance of Steady-state Hybrid Scenario with Strike Point on Tungsten Rings

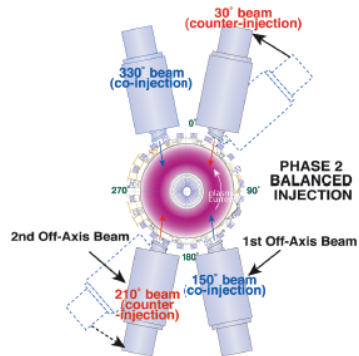
- Hybrid discharges feature broad current profile maintained by internal MHD
- β_N and energy confinement equivalent to all-carbon divertor shot
- ECCD applied on-axis
- No increase in core radiation



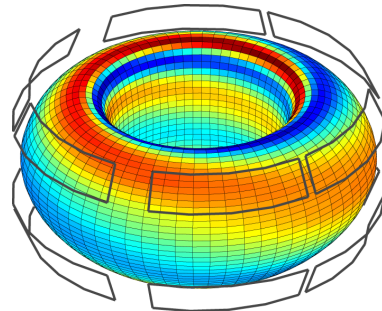
B.S. Victor/APS Conference/Oct. 2017

DIII-D Research Will Provide a Scientific Foundation for Mitigating the Risk of Uncontrolled Transients

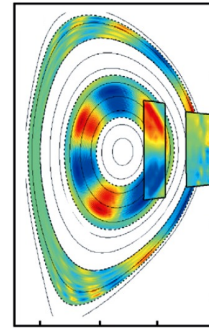
- Unique capabilities support science of disruption avoidance and mitigation



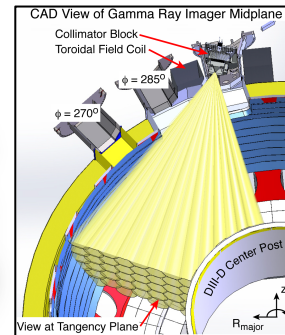
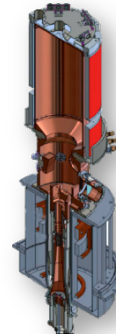
Rotation Control



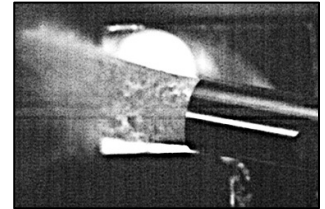
MHD simulation



NTM control

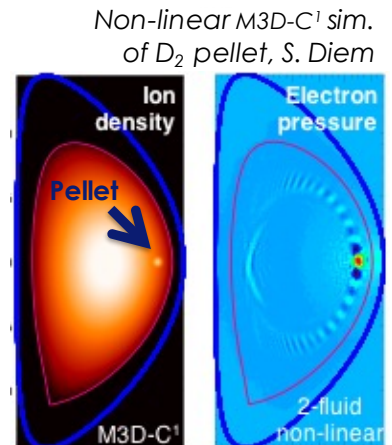
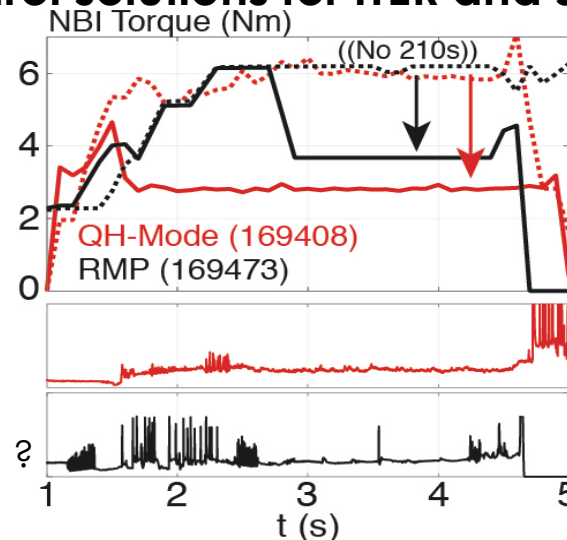
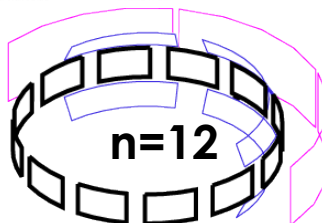
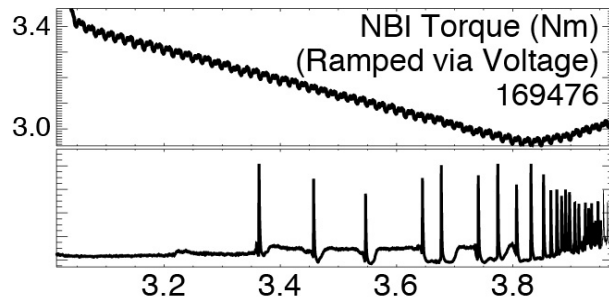


Diagnostics



Mitigation: SPI & shell pellets

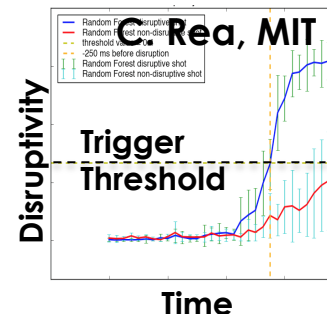
- Explore and optimize ELM control solutions for ITER and steady state tokamaks



Disruption Mitigation Research Aims to Provide Robust Mitigation in ITER and Steady-State Reactor

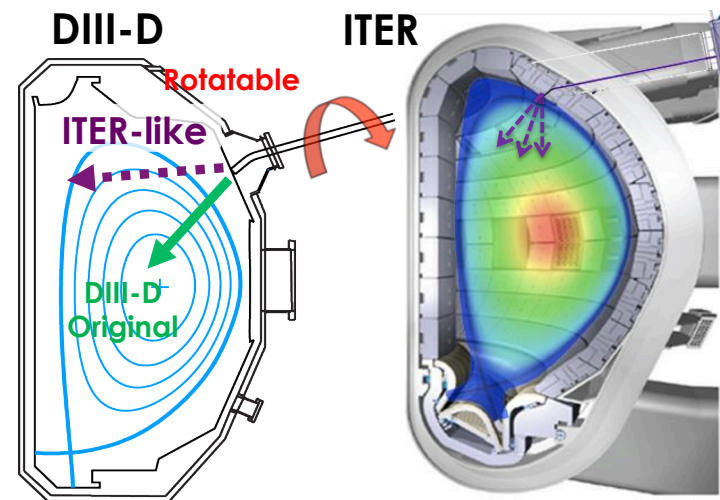
Disruption Avoidance is critical to meeting ITER or reactor goals

- “Active” MHD spectroscopy/NBI feedback to *avoid stability boundaries*
- *Active stabilization*: RMP feedback + synchronized ECCD
- *Real-time Prediction*: Machine learning algorithms →
- Off-normal supervisor algorithms for *integrated control*



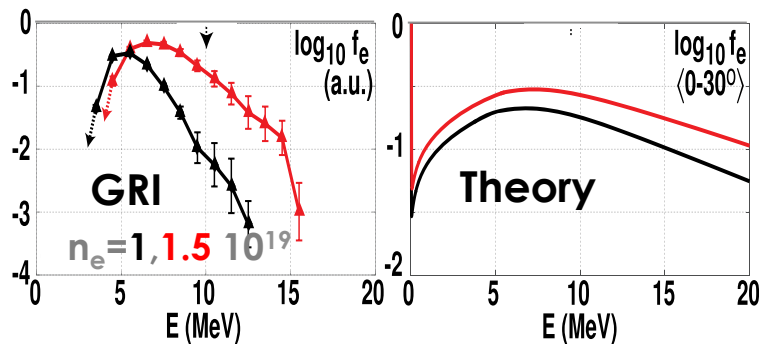
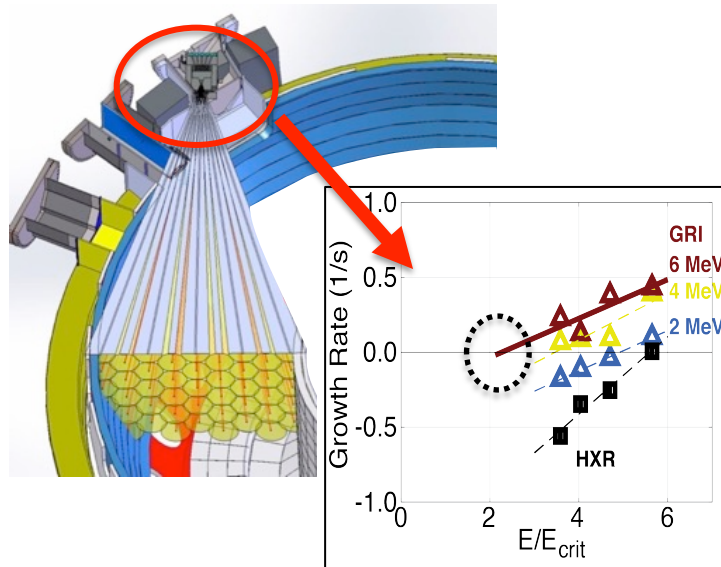
Disruption Mitigation: Testing ITER-prototype shattered pellet injectors

- Injection Geometry: ITER-like shallow trajectory reduces SPI performance (*similar to MGI*)
- Now injecting multiple shattered pellets



Improved Understanding of Runaway Electron Generation and Dissipation Informs Mitigation Development

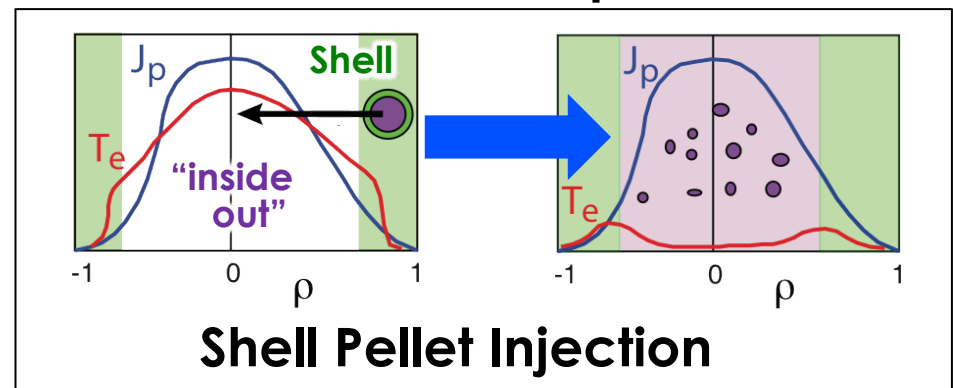
Gamma-ray Imager: Physics of RE growth and rapid dissipation



Synchrotron imaging shows RE interplay with MHD



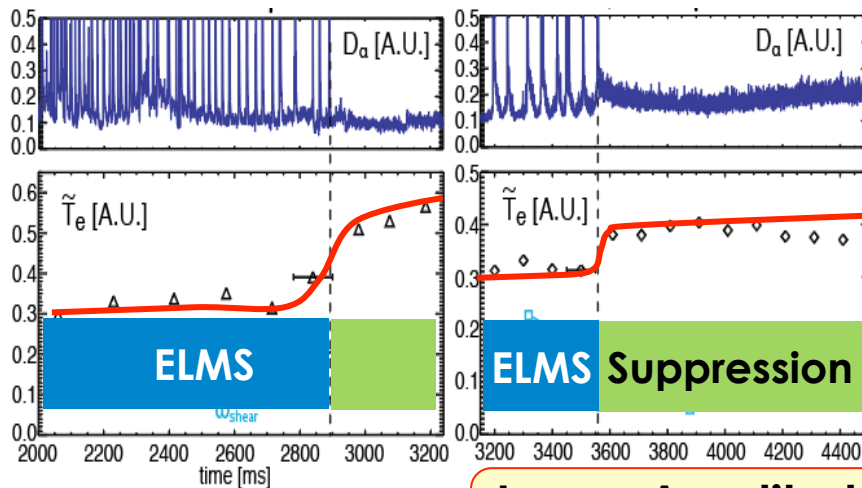
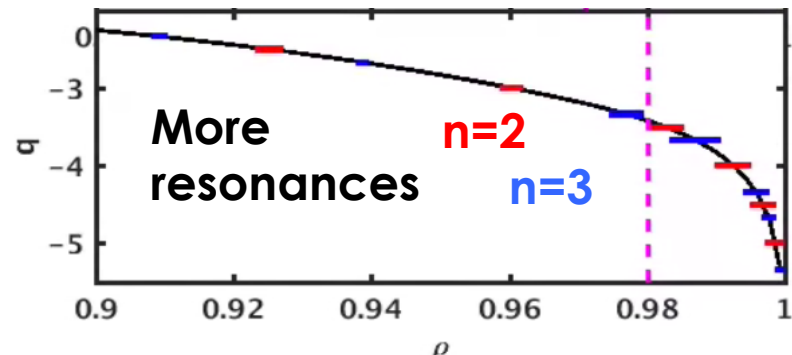
Test advanced concepts: Inside-Out



New Power Supply from ASIPP Enabling Key Insights on ELM Suppression Physics



Expands spectral flexibility significantly
Reduced RMP threshold with $n=2 + n=3$

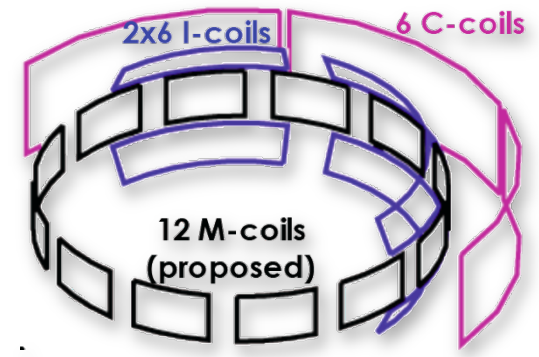
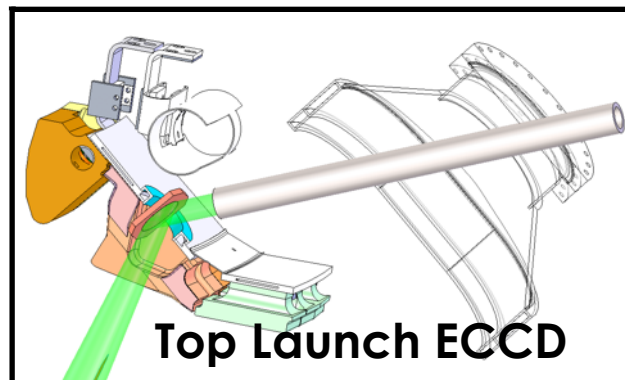
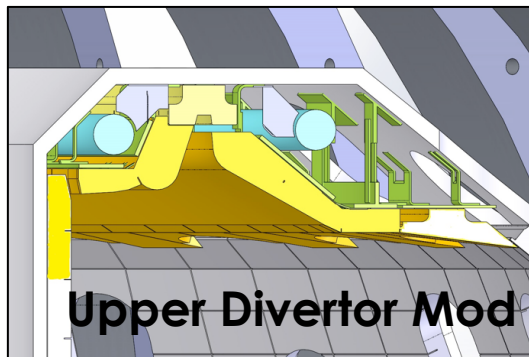


Fluctuation measurements are consistent with reduced pedestal gradients during ELM-free RMP and QH-mode operation

Large-Amplitude ELM Suppression Achieved in Fully Non-inductive Plasmas Relevant to the ITER SS Mission

Planned Upgrades Leverage Existing Capabilities To Advance Scientific Basis for Steady State Tokamaks

Scientific Exploration	Enabled by
Divertor model validation & optimization	Divertor mods and diagnostics
SS High beta, high bootstrap current	Co-counter Off-axis beam
Reactor-relevant current drive schemes	Top-launch EC, Helicon, HFS-LHCD
Reactor-relevant materials	Material Exposure, Migration
Electron heated regimes	10 gyrotron system
3d physics spectral flexibility ($n=1-4$)	New 3d coils and power supplies

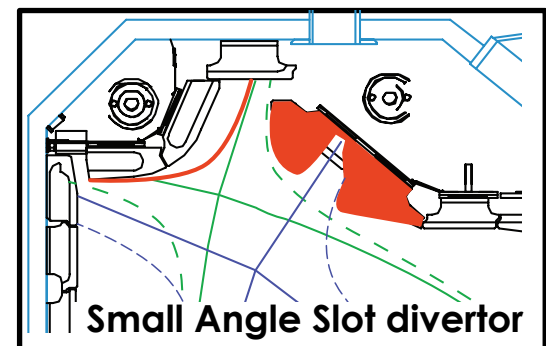
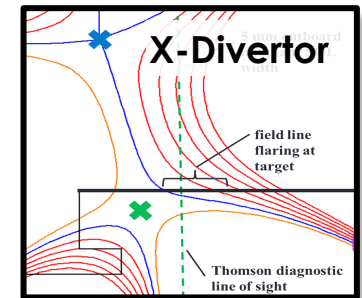
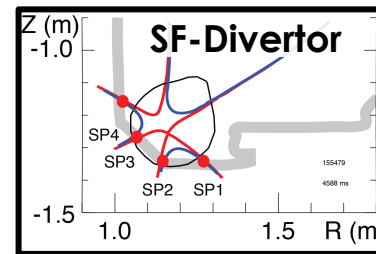
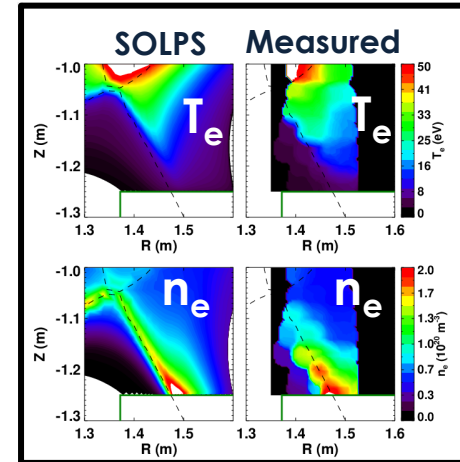
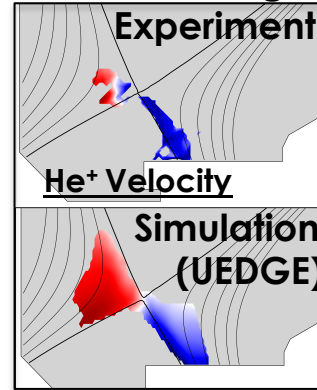


DIII-D Will Develop a Scientific Basis for Boundary Solutions Needed for Steady-State Reactors

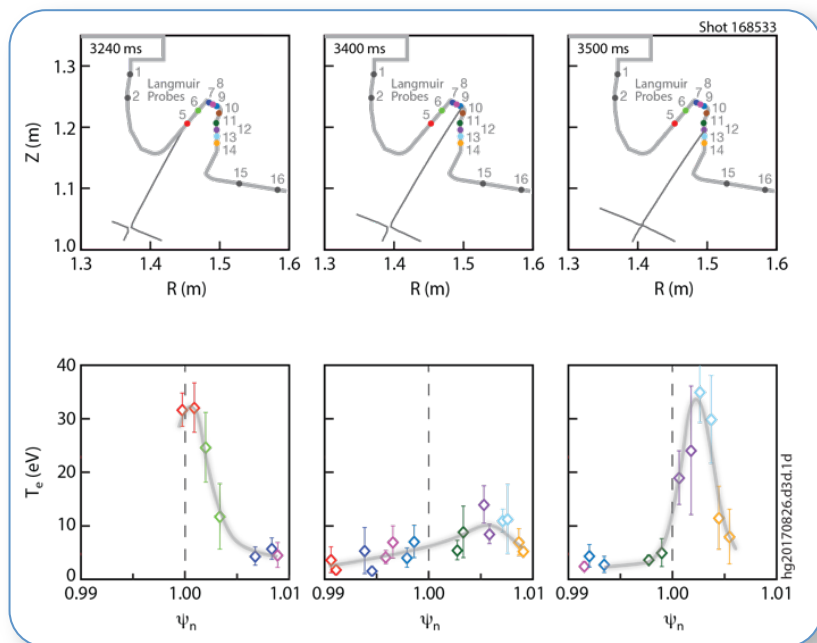
- Advance scientific understanding and predictive capability through **key measurements** and **extensive model validation**
 - Determine key dissipation processes
 - Resolve role of drifts and turbulence
 - Quantify role of neutrals and benefits of closure
 - Elucidate magnetic topology effects
- Develop advanced divertors compatible with high performance
 - Maximize heat flux dissipation without degrading core
 - Staged divertor concept tests

Close interaction with simulation efforts: prediction and data analysis

2D Flow Imaging

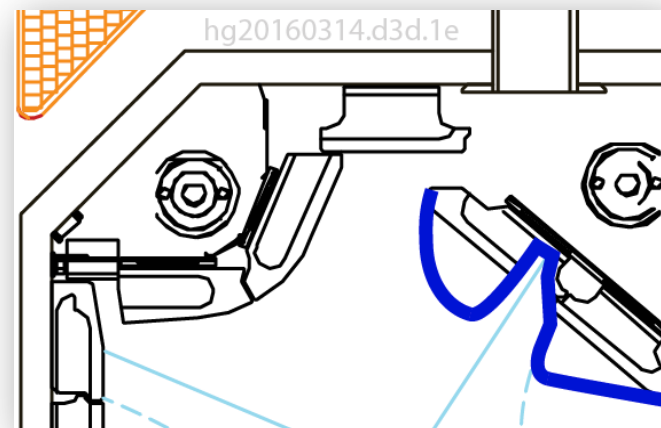


Phase I of Planned Divertor Modifications Provided Data To Compare With SOLPS Simulation

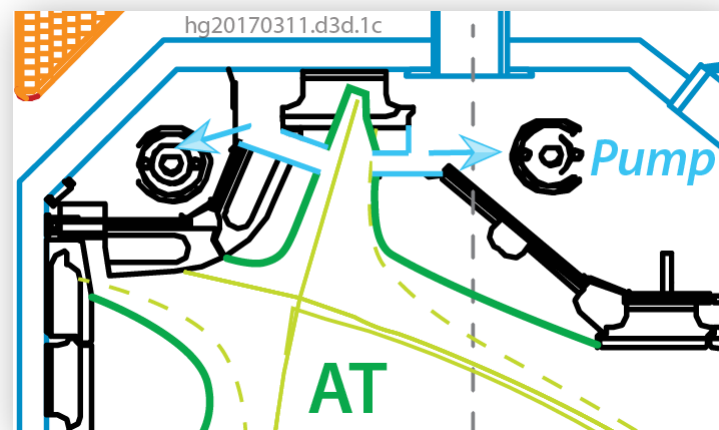


- Baffled divertor with slanted sides traps recycled neutrals
- Reduced T_e and ion flux compared to open divertor
- Phase II will provide pumping and shape to integrate with AT core

SAS I (FY17)



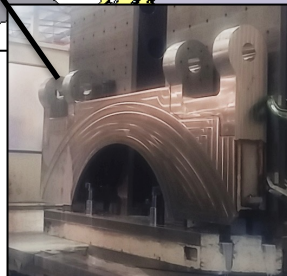
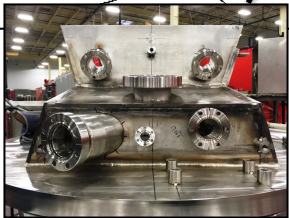
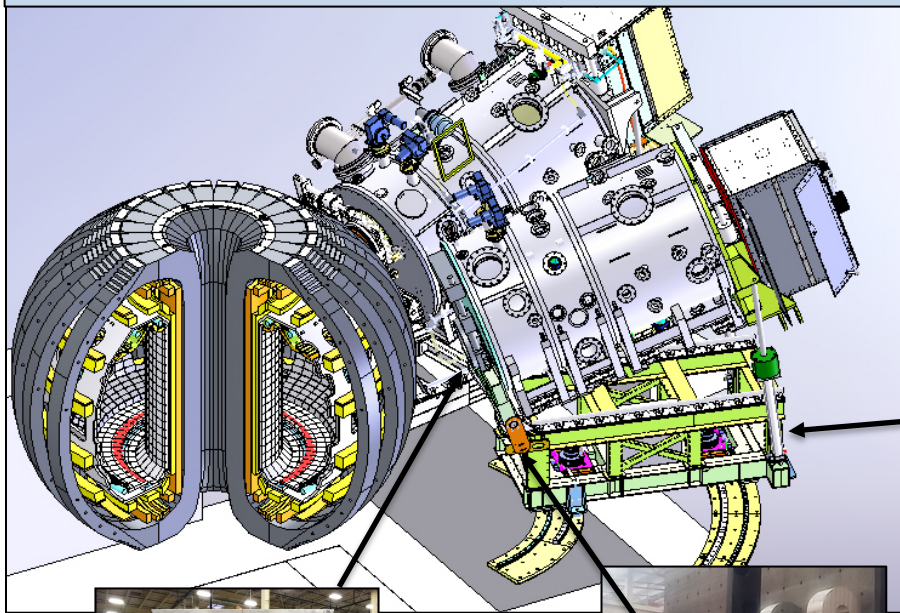
SAS II (FY21)



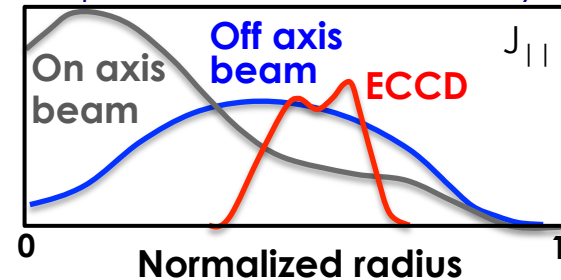
DIII-D Utilizing Flexible Heating and Current Drive Systems to Develop Path to High β Steady State

- Move tokamak solutions from **peaked** to **broad** current profiles
 - $\beta_N=5$ potential with low disruptivity (High q_{\min} scenarios)

210 Co-Counter Off-Axis Beam Design



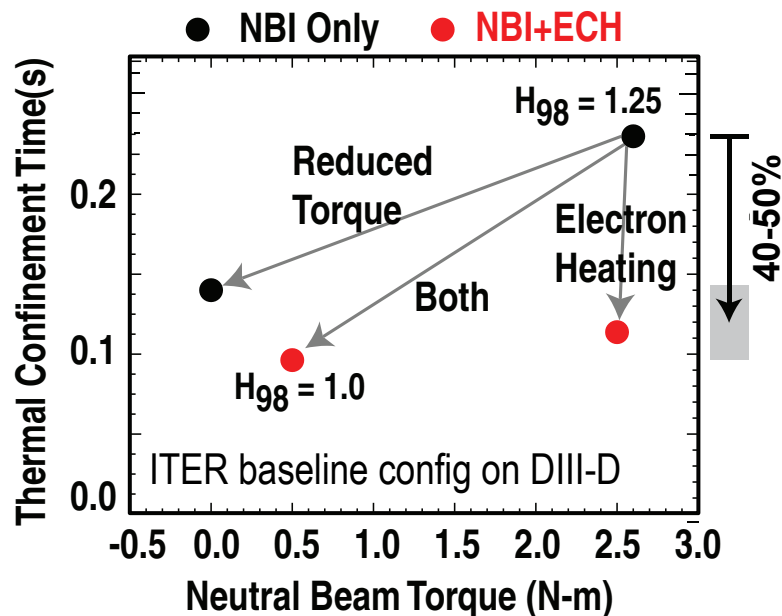
Required tools for stationarity:



Install during one-year shutdown (May 2017-2018)

Increased ECH Power Is a Critical Element to Advancing Studies Towards Reactor-Like Conditions

- **Increased off-axis ECH power and balanced NBI to**
 - Access low rotation, dominant electron heated regimes
 - Provide tearing mode control for disruption avoidance

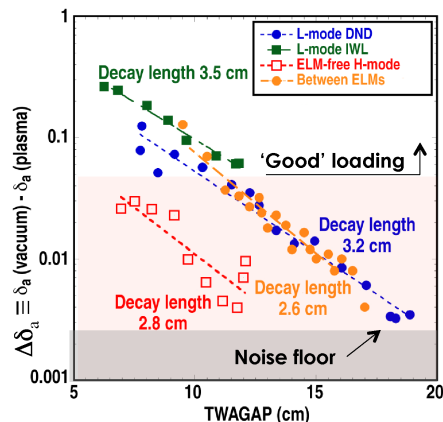
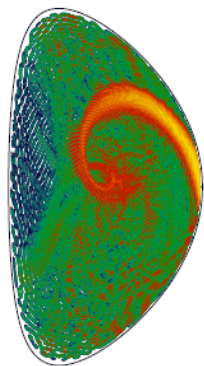
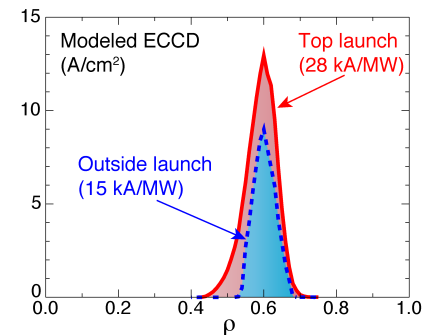
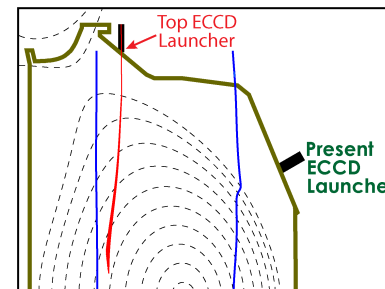


**Completing
installation of
new 1.5MW
117.5GHz
CPI gyrotron**

- **Replacement gyrotrons (3)
(1MW 110GHz) CPI tubes**
 - 10+ year productive lifetimes
- **Expand system to advance
AT and divertor program
goals (10 tubes total)**

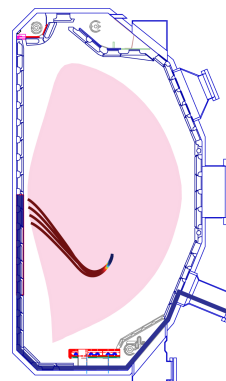
DIII-D Proceeding with Tests of Three Transformational Current Drive Technologies

- Top launch ECCD doubles efficiency



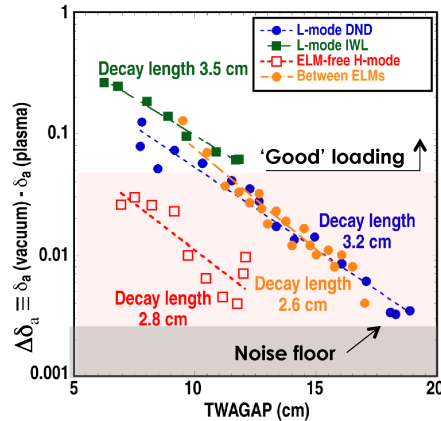
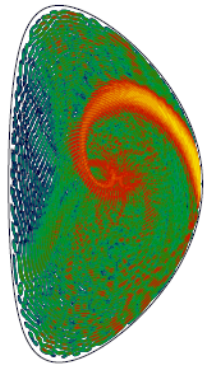
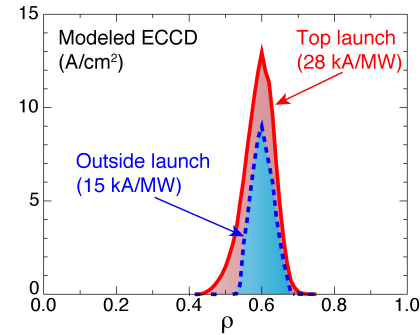
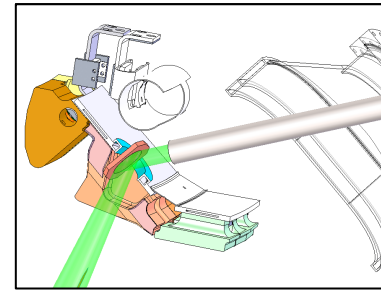
- Helicon ready to test physics of coupling at high power

- HFS LHCD in development



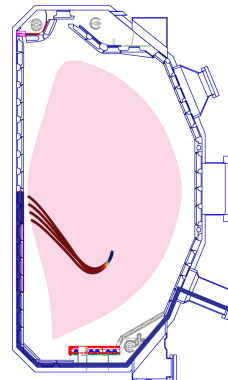
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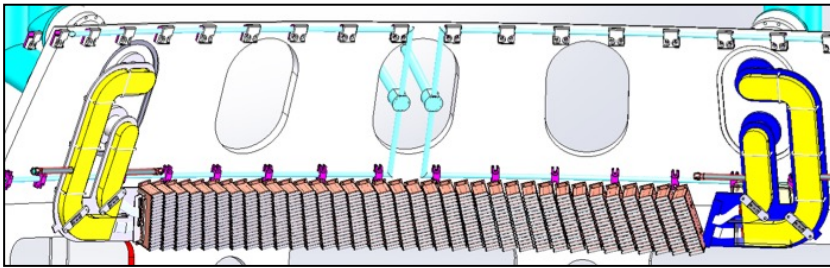
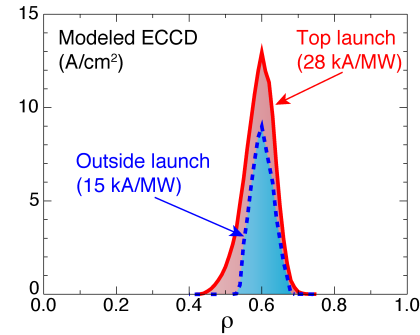
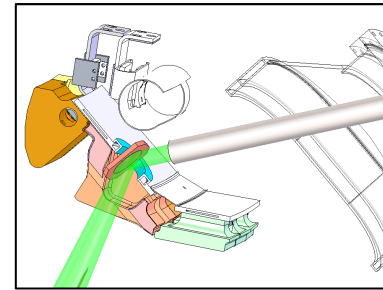
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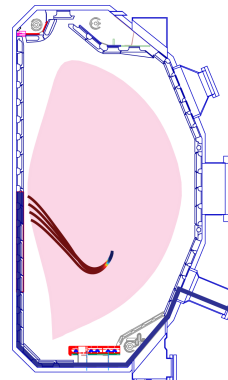
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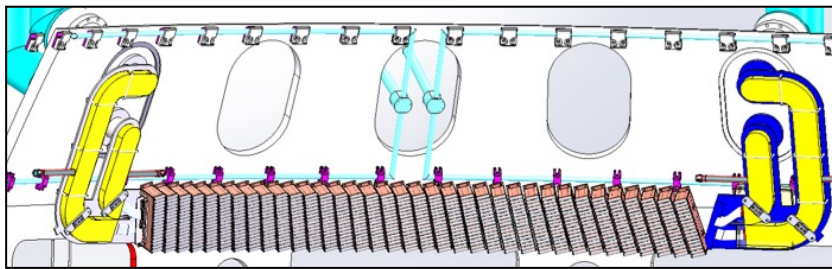
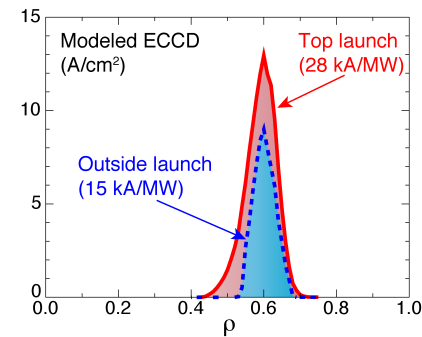
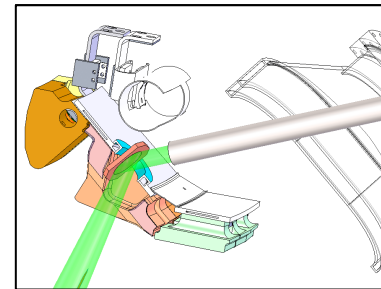
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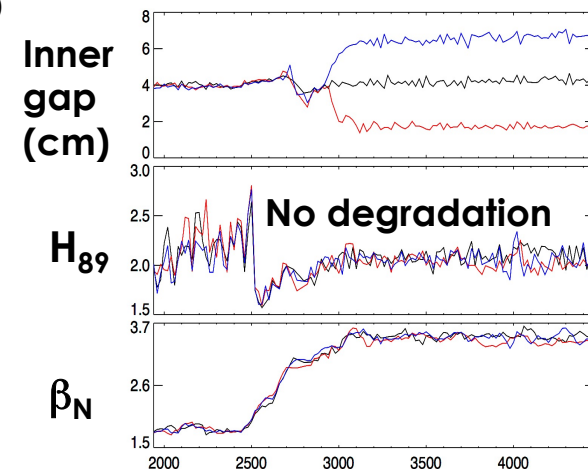
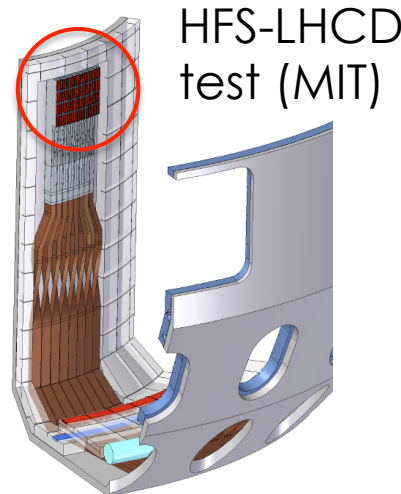
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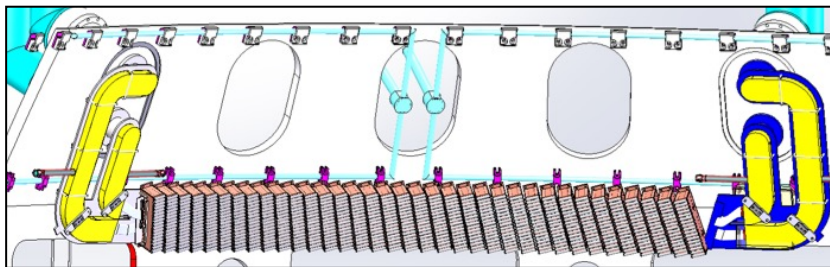
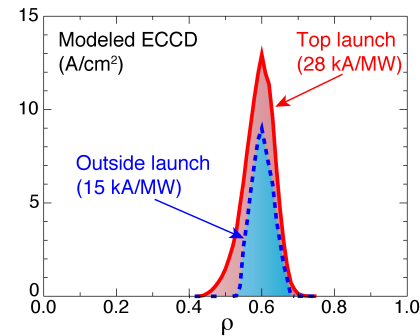
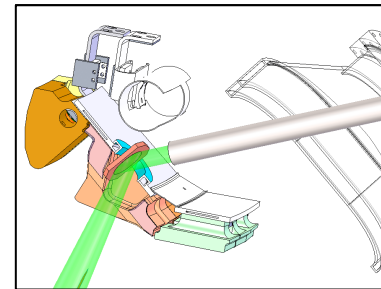
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 - AT plasmas found compatible with low inner gaps →



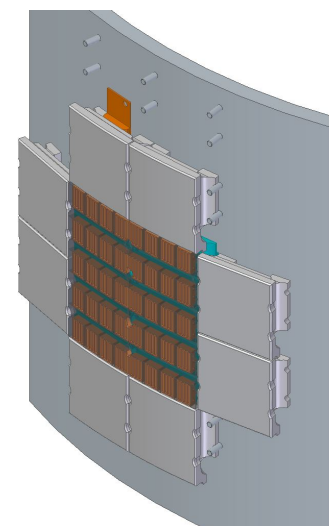
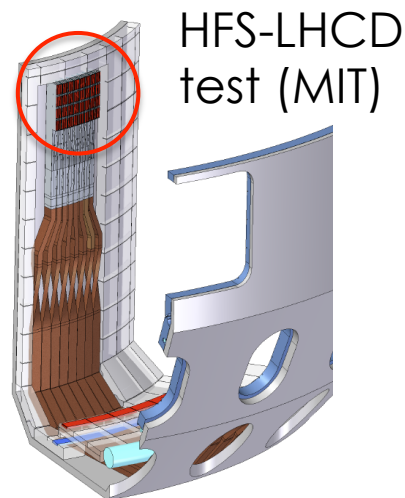
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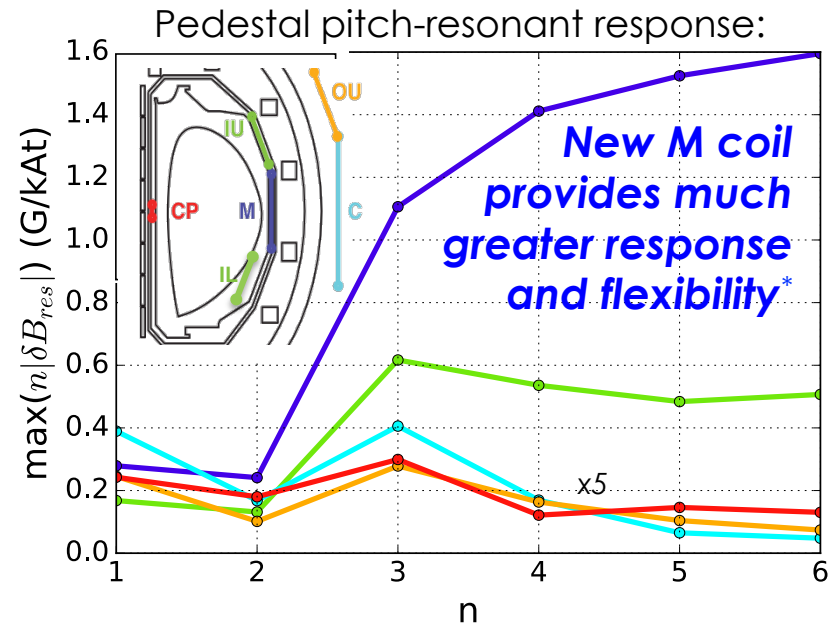
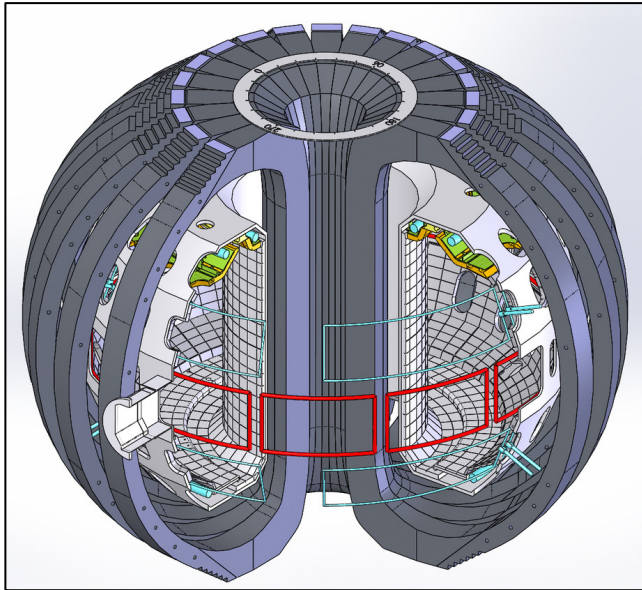


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- **HFS LHCD in development**
 - AT plasmas found compatible with low inner gaps →
 - Possible HFS test tile in FY18, completion after LTO3



New M-Coil and 2nd Programmable Power Supply Expands Capability for 3D Studies and Transient Control



- **Power supply enables control of**
 - All 18 PF coils independently for advanced 2D shaping & divertor
 - Each I coil independently, Tor+pol spectrum simultaneously
- **New 12 coil Midplane array opens vital new research capabilities***
 - Stronger drive for NTV and n=3 & 4 rotation, spectrum optimization

DIII-D Is Advancing the Scientific Basis for Magnetic Fusion Energy Supporting Successful Operation of ITER

DIII-D results are providing a strong foundation for successful ITER operation and its scientific exploitation

DIII-D is addressing foundational scientific questions for developing validated predictive simulation

New DIII-D capabilities will enable significant steps toward developing the solutions needed for future steady-state tokamak fusion reactors